Énergie NB Power

SU-2

POINT LEPREAU NUCLEAR GENERATING STATION

Annual Compliance Report

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Document Approval

The following signatures are required prior to issue of this document.

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Classification Statement

Proprietary usage

Executive Summary

This report describes the 2015 results of the environmental protection program for the Point Lepreau Nuclear Generating Station (PLNGS).

In 2015, 1258 samples were analysed to monitor environmental radiation around Point Lepreau and across the province in general. There were 253 other samples, including 147 Quality Assurance (QA) samples.

The analyses indicate that radiation dose from PLNGS emissions continues to be well below the public dose limit (1000 microsieverts per annum), and also well below the design and operating target for PLNGS (50 microsieverts per annum).

Source of Dose to the Representative Person	***Individual Dose ($\mu Sv \cdot a^{-1}$)***
PLNGS airborne emissions	0.51
PLNGS liquid emissions	0.05

Reports are issued to other regulators for non-radioactive hazardous emissions. These reports are described in this report.

Alignment to the Canadian Standards Association (CSA) standards *N288.4-10, Environmental monitoring programs at Class I nuclear facilities and uranium mines and mills* and *N288.5-11, Effluent monitoring programs at Class I nuclear facilities and uranium mines and mills* was progressed in 2015. The following were in the review stage at the end of 2015:

- Ecological risk assessment report (*Point Lepreau Generating Station- Site Wide Risk Assessment: Human Health and Ecological Risk Assessment, February 2016, Arcadis Canada Inc)* as per *N288.6-12, Environmental risk assessments at class I nuclear facilities and uranium mines and mills*).
- Fish entrainment report (*Point Lepreau Generating Station- Final: Entrainment Monitoring Plan and Implementation for Point Lepreau Generating Station, March 2016, Arcadis Canada Inc.*).
- Fish impingement report (*NB Power- Progress Report Impingement Monitoring at Point Lepreau Generating Station 2013-2014, March 2016, Arcadis Canada Inc*.).

A contract has been awarded to Canadian Nuclear Laboratories (CNL) to close the gaps and implement the standards.

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1 Introduction

This document describes the results of the Radiation Environmental Monitoring Program (REMP) and summarizes the reports for non-radioactive hazardous emissions for the year 2015.

The REMP is described in *IR-03541-HF02, Radiation Environmental Monitoring Program (REMP).* The requirement for the REMP is stated in *SI-01365-A108, Radiation Protection Directives,* and *SR-79100, Solid Radioactive Waste Management Facility 2007 Safety Report*. The underlying reason for the program is the large inventory of radionuclides that are present onsite. The program operates in conjunction with *SI-01365-L20, Online Monitoring and Control of Liquid and Airborne Effluents,* a program which monitors and controls effluents at their source. The Derived Release Limits (DRLs) are calculated in *RD-01364-L1, Derived Release Limits for Radionuclides in Airborne and Liquid Effluents*.

As part of its overall Management System, PLNGS has an Environmental Management System (EMS) (*SI-01365-P101 Developing and Maintaining the Environmental Management System (EMS))* in place that is registered to National Standards of Canada, *CAN/CSA-ISO 14004-2004-04 Environmental Management Systems – General Guidelines on Principles, Systems and Support Techniques* (2nd Edition). All activities and products that could impact the environment have been identified and logged in a database. From this database, a list of significant environmental aspects (SEAs) was developed and it forms the foundation for the EMS. Management programs are in place for each of the SEAs to ensure compliance with the standards. The SEAs include radiological and non-radiological releases to water and air, waste management and accident management. Environmental assessment and improvement programs have been developed for the SEAs to ensure continual improvement.

All activities that support PLNGS are controlled by the PLNGS Management System. The environmental radiation monitoring program falls under the primary process *PRR-00660-SU-2 SU-02 Provide Environmental Services*. All sub-processes related to routine environmental radiation monitoring come under SU-02.

All radionuclide analyses in 2015 were performed in the Fredericton Health Physics Laboratory at 420 York Street, Fredericton, NB.

The basis of the REMP complies with National Standards of Canada, *CAN/CSA-N288.4-M90 (R2008) Guidelines for Radiological Monitoring of the Environment)*. Since this standard was replaced in 2010 with CSA standard *N288.4-10 Environmental monitoring programs at Class I nuclear facilities*, the REMP will be modified to comply with the new standard by 2018.

The Radiation Environmental Monitoring Program for PLNGS fulfils several objectives. These are to:

1) permit the estimation of dose to the Representative Person and populations from the radioactive emissions from PLNGS and its Solid Radioactive Waste Management Facility (SRWMF). This estimation of dose is achieved through the analyses of environmental and effluent samples.

2) provide data to confirm compliance of PLNGS and the SRWMF with release guidelines and regulations and to provide public assurance of compliance. These provisions are achieved through the issuance of the annual report to all interested parties.

3) establish and maintain the capability for environmental monitoring so that an effective response can be made to emergency conditions. This response is assured by maintaining the resources to step up the monitoring program during increased emissions that are only likely during an accident. The ability to interpret the data and make recommendations is also maintained.

4) maintain a database to facilitate the detection of trends. The database is maintained by storing all results on a computer system that has the capability of reporting and graphing any desired subsets of the data.

5) verify or refine environmental models used in the calculation of Derived Release Limits (DRLs). Verification is achieved by comparing the theoretical dispersion factor with one calculated empirically. In addition, other exposure routes to the public are continually evaluated.

6) determine the fate of released radioactive materials to show whether any pathway to humans has been overlooked. The deposition of radioactive material is determined through the collection and analysis of sample media outside of the established program. In addition, any results that are not consistent with effluent results are investigated.

The capability of the radiation monitoring laboratory is assessed through the QA program and through the daily analytical checks. These checks demonstrate the accuracy and consistency of analyses.

The following sections will briefly describe the program. Details are provided on PLNGS emissions, results of analyses, dose estimates, and the quality assurance program.

2 PLNGS Radioactive Emission Data

Emissions from PLNGS continue to be at low levels as indicated in Table 2.01. By the time these emissions reach the edge of the exclusion zone, they are diluted below the detection limits of most analytical procedures.

Table 2.01: Radionuclides Detected in Effluents

Nuclide	***Gaseous Effluent DRL ($Bq \cdot a^{-1}$)***	***Emission (Bq)***	***DRL (%)***	***Liquid Effluent DRL* ($Bq \cdot a^{-1}$)***	***Emission (Bq)***	***DRL (%)****
H-3	2.8E+17	1.4E+14	4.8E-02	4.6E+19	1.4E+14	3.0E-04*
C-14	6.8E+15	7.1E+10	1.0E-03	3.2E+14	1.0E+10	3.1E-03*
Ar-41	2.6E+17	4.0E+12	1.5E-03	-----------	-----------	-----------
Sc-46	-----------	-----------	-----------	-----------	2.5 E +05	-----------
Cr-51	-----------	-----------	-----------	1.6E+16	8.0E+06	5.1E-08*
Mn-54	-----------	-----------	-----------	8.0E+13	7.9E+05	9.8E-07*
Fe-59	-----------	-----------	-----------	3.1E+12	1.6E+07	5.1E-04*
Co-60	-----------	-----------	-----------	3.8E+13	1.5E+08	4.0 E-04*
Zn-65	-----------	-----------	-----------	9.7E+12	7.9E+05	8.2E-06*
Kr-85m	2.3E+18	5.7E+09	2.4E-07	-----------	-----------	-----------
Kr-87	4.1E+17	1.5E+10	3.7E-06	-----------	-----------	-----------
Kr-88	1.2E+17	2.7E+11	2.2E-04	-----------	-----------	-----------
Sr-90	-----------	-----------	-----------	6.0E+15	1.2E+05	2.0E-09*
Nb-94	-----------	-----------	-----------	3.7E+14	1.5E+05	4.0E-08*
Zr-95	-----------	-----------	-----------	8.6E+13	8.4E+07	9.8E-05*
Nb-95	-----------	-----------	-----------	8.6E+14	1.7E+08	1.9E-05*
Nb-97	-----------	-----------	-----------	-----------	5.7E+08	-----------
Sn-113	-----------	-----------	-----------	4.1E+12	3.1E+06	7.5E-05*
Sb-122	-----------	-----------	-----------	9.4E+14	1.3E+06	1.4E-07*
Sb-124	-----------	-----------	-----------	5.2E+14	2.5E+07	4.9E-06*
Sb-125	-----------	-----------	-----------	1.3E+15	5.6E+06	4.3E-07*
Xe-131m	4.3E+19	5.8E+09	1.4E-08	-----------	-----------	-----------
Xe-133	1.1E+19	5.8E+10	5.0E-07	-----------	-----------	-----------
Xe-135	1.5 E+18	6.7 E+10	4.6 E-06	-----------	-----------	-----------
Xe-135m	8.8 E+17	5.1 E+10	5.8E-06	-----------	-----------	-----------
Cs-137	-----------	-----------	-----------	4.7E+14	1.1E+05	2.4E-08*
Xe-137	-----------	1.5E+11	-----------	-----------	-----------	-----------
Xe-138	2.9E+17	1.8E+11	6.2E-05	-----------	-----------	-----------
Gd-153	-----------	--------	-----------	4.2E+15	1.6E+07	3.9E-07*
Tb-160	-----------	-----------	-----------	6.4E+14	9.5E+07	1.5E-05*
Alpha	-----------	-----------	-----------	-----------	6.7E+06	-----------
Beta	-----------	-----------	-----------	-----------	5.5E+07	-----------
Total			5.1E-02	Total		4.5E-03

* To calculate %DRL for emissions from some locations and during outages, an adjustment is made to compensate for different flow rates.

3 Sample Media, Locations and Frequencies (REMP)

The data contained in this report are for samples collected from January 1 to December 31, 2015, with some overlap for air, precipitation and thermo luminescent dosimeter (TLD) samples. During this time, the major media analysed and their frequency of collection were as indicated in Table 3.01. Sample collection usually takes place at least once each week throughout the year. The number of each sample type collected in 2015 and the major radionuclide measurements performed on that sample type are listed in Table 3.02.

The miscellaneous sample group includes those samples that are above and beyond the listed categories or are not routinely collected. Miscellaneous samples include source leak testing and contamination monitoring.

The major sample locations are listed in Table 3.03 (details in Appendix C) and shown in Figures 3.01 to 3.05. Each "Indicator" site has a three or four-character identification code (e.g., F01, I10A). An Indicator site is one within the possible influence of PLNGS emissions. A "Reference" site is outside the influence of PLNGS emissions and is identified by the letter R at the end of the location code (e.g., A01R).

Sample locations for mobile seafood species (lobster, fish, etc.) caught in the Lepreau area are specified as accurately as reasonably possible. The location of capture, however, may bear little relationship to where the animal has been in the recent past. The availability of such samples is not generally predictable and is outside of the control of the laboratory.

Table 3.01: Schedule of Sample Collection and Analysis

Sample Medium	*Typical Frequency*
Atmospheric Sampling	
Airborne Particulates	Monthly (integrated sample)
Airborne Iodines	Monthly (integrated sample)
Water Vapour	Monthly (integrated sample)
Carbon Dioxide	Monthly (integrated sample)
Ambient Gamma Measurements (TLDs)	Quarterly (integrated sample)
Gaseous Effluent Monitor (GEM) Particulates	Weekly Composite (integrated sample)
Terrestrial Sampling	
Ambient Gamma Measurements (TLDs)	Quarterly (integrated sample)
Milk - commercial dairy - dairy farms	Monthly Quarterly
Well Water	Semi-annually
Pond, Puddle and Surface Water	Quarterly
Berries	Weekly in Season
Garden Vegetables	Weekly in Season
Vegetation	Monthly
Soil	Quarterly
Monitoring Well Water (Near Plant)	Annually
Precipitation	Monthly (integrated sample)
Marine Sampling	
Seawater	Quarterly
Clams	Quarterly When Available
Fish	Quarterly When Available
Lobster	Quarterly When Available
Periwinkles	Monthly When Available
Aquaculture Salmon	Quarterly When Available
Scallops	Quarterly When Available
Crabs	Quarterly When Available
Dulse	Monthly When Available
Other Sea Plants	Quarterly
Sediment	Quarterly
Ambient Gamma Measurements of Intertidal Zone (Ion Chamber)	Quarterly
Liquid Effluent Monitor (LEM) Composite Water	Monthly Composite (integrated sample)

Table 3.01 (continued): Schedule of Sample Collection and Analysis

Sample Medium	***Typical Frequency***
Solid Radioactive Waste Management Facility	
Bore Hole Water	Three Times Per Year
Parshall Flume Water	Weekly
Ambient Gamma Measurements (TLDs)	Quarterly (integrated sample)
Hemlock Knoll Regional Sanitary Landfill	
Ambient Gamma Measurements (TLDs)	Quarterly (integrated sample)

Table 3.02: Sample Information

Sample Medium	*Number of Samples*	*Radionuclide Measurements*
Atmospheric Sampling		
Airborne Particulates	89	gamma emitters & gross alpha/beta
Airborne Iodines	89	Iodine-130,131,132,133,135
Water Vapour	86	Tritium
Carbon Dioxide	35	Carbon-14
Ambient Gamma Measurements (TLDs)*	99*	gamma exposure
GEM Particulates	52	Strontium-89,90 & gamma emitters
Terrestrial Sampling		
Ambient Gamma Measurements (TLDs)*	99*	gamma exposure
Milk - commercial dairy - dairy farms	12 12	gamma emitters & tritium
Well Water	25	gamma emitters, gross alpha/beta & tritium
Pond, Puddle and Surface Water	34	gamma emitters & tritium
Berries	3	gamma emitters
Garden Vegetables	20	gamma emitters
Vegetation	13	gamma emitters
Soil	28	gamma emitters
Monitoring Well Water (Near Plant)	11	gamma emitters & tritium
Precipitation	29	gamma emitters & tritium
Marine Sampling		
Seawater	17	gamma emitters & tritium
Clams	5	gamma emitters
Fish	7	gamma emitters
Lobster	6	gamma emitters
Periwinkles	11	gamma emitters
Aquaculture Salmon	2	gamma emitters
Scallops	2	gamma emitters
Crabs	0	gamma emitters
Dulse	7	gamma emitters
Other Sea Plants	13	gamma emitters
Sediment	46	gamma emitters
Ambient Gamma Measurements of Intertidal Zone (Ion Chamber)	36	gamma exposure
LEM Composite Water	12	Strontium-89,90, gamma emitters, gross alpha/beta

*The same TLD measures gamma dose from radionuclides in the air and on the ground.

Table 3.02 (continued): Sample Information

Sample Medium	*Number of Samples*	*Radionuclide Measurements*
Solid Radioactive Waste Management Facility		
Bore Hole Water	100	gamma emitters & tritium
Parshall Flume Water	160	gamma emitters & tritium
Ambient Gamma (TLDs)	176	gamma exposure
Hemlock Knoll Regional Sanitary Landfill		
Ambient Gamma (TLDs)	16	gamma exposure
Other		
Miscellaneous	89	as required
Quality Assurance	314	as scheduled

Table 3.03: General Location Codes

Code	***Location***
A	West of Pennfield Ridge
B	Pennfield to New River Beach (inclusive)
C	Lepreau and Lepreau Harbour
D	Little Lepreau and Little Lepreau Basin
E	Maces Bay
F	Welch Cove
G	Pt. Lepreau lighthouse and surrounding area
H	Duck Cove
I	PLNGS site – northeast quadrant
J	PLNGS site – southeast quadrant
K	PLNGS site – southwest quadrant
L	PLNGS site – northwest quadrant
M	PLNGS
N	Dipper Harbour
P	East of Dipper Harbour East to Musquash
Q	Lorneville
S	Saint John and surrounding area
T	Taymouth
X	Fredericton and surrounding area
Y	Hemlock Knoll Regional Sanitary Landfill

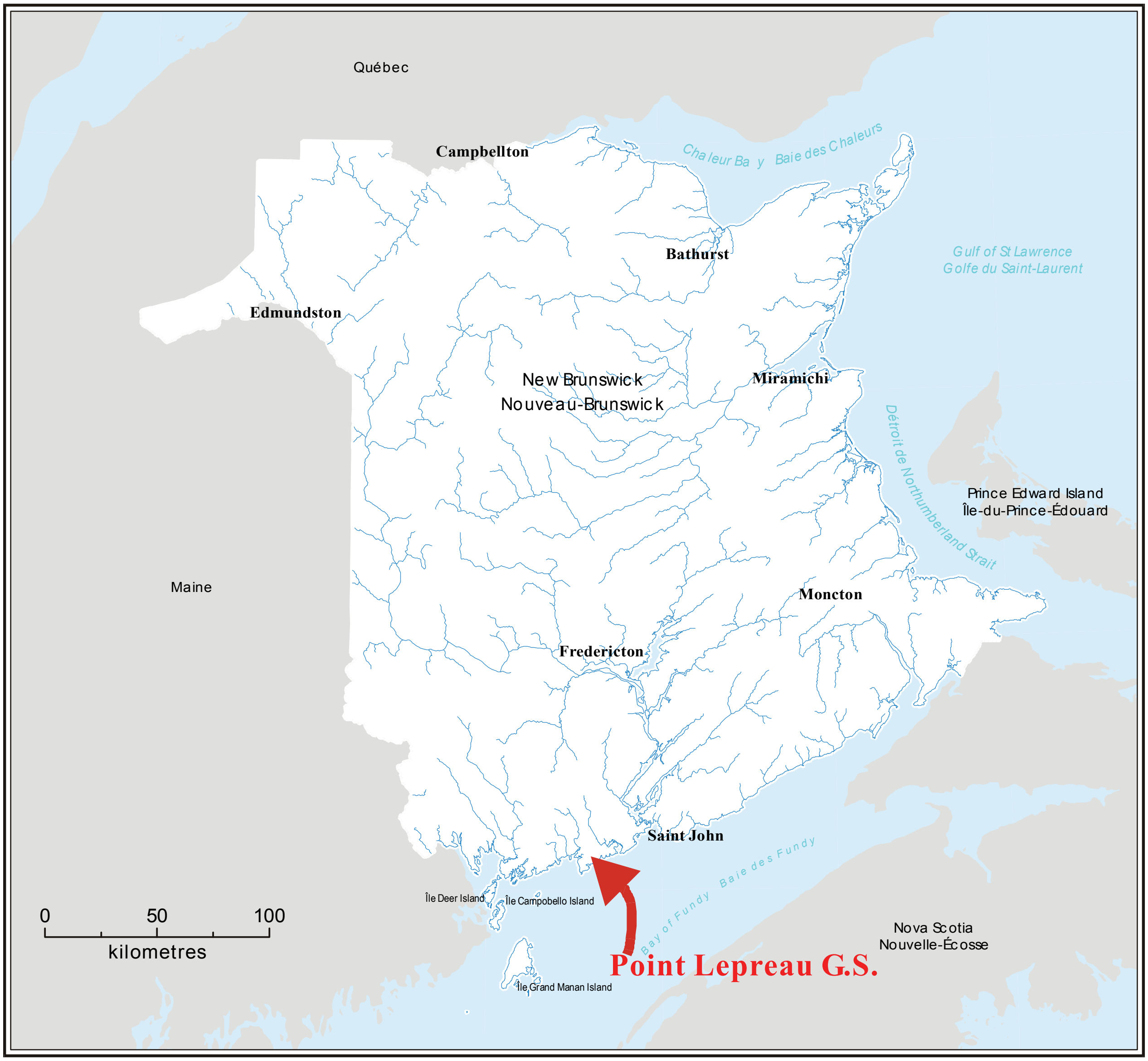


Figure 3.01: Map of New Brunswick

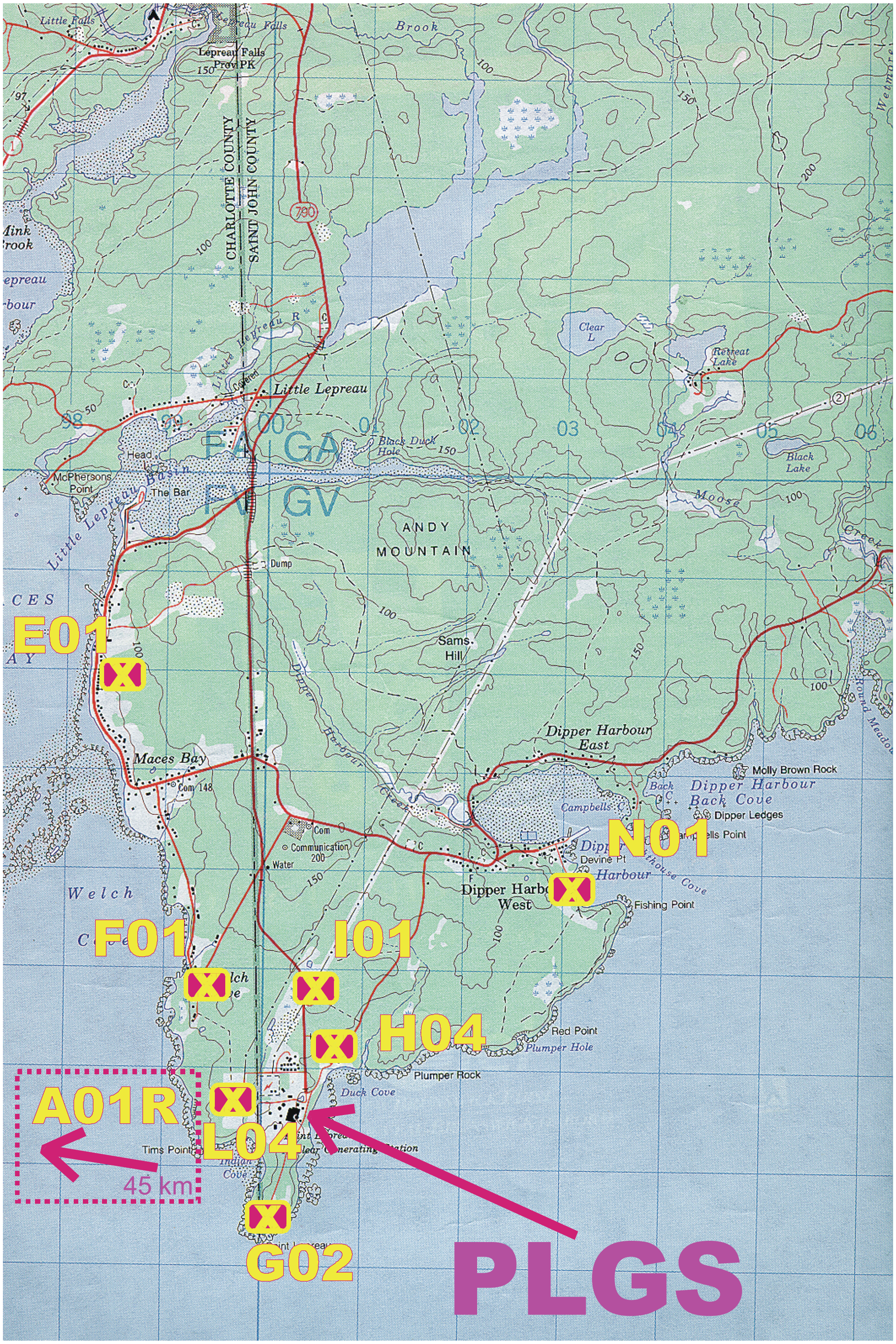


Figure 3.02:   Air Monitoring Stations

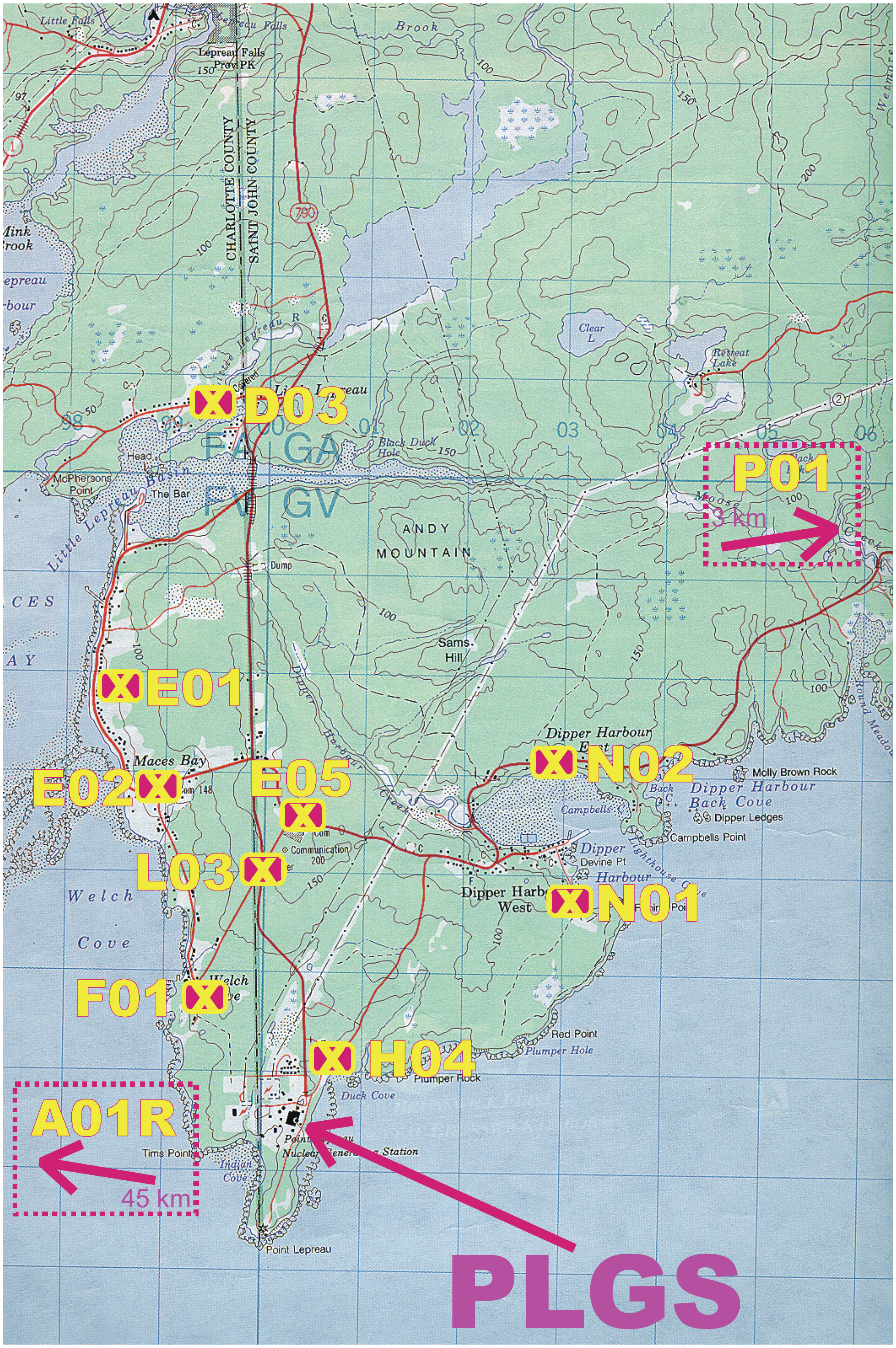


Figure 3.03: Well Water Sites

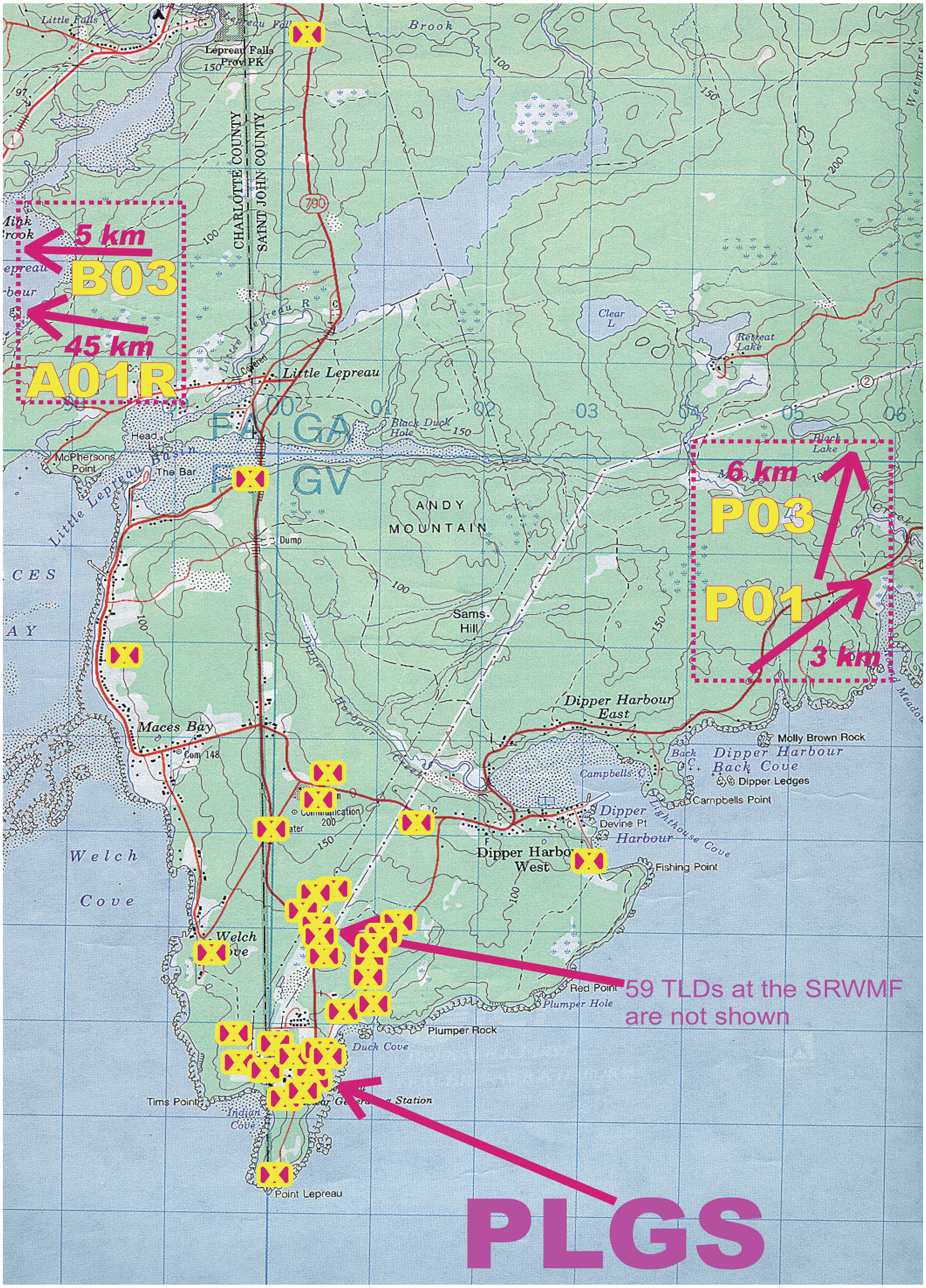


Figure 3.04: TLD Sites

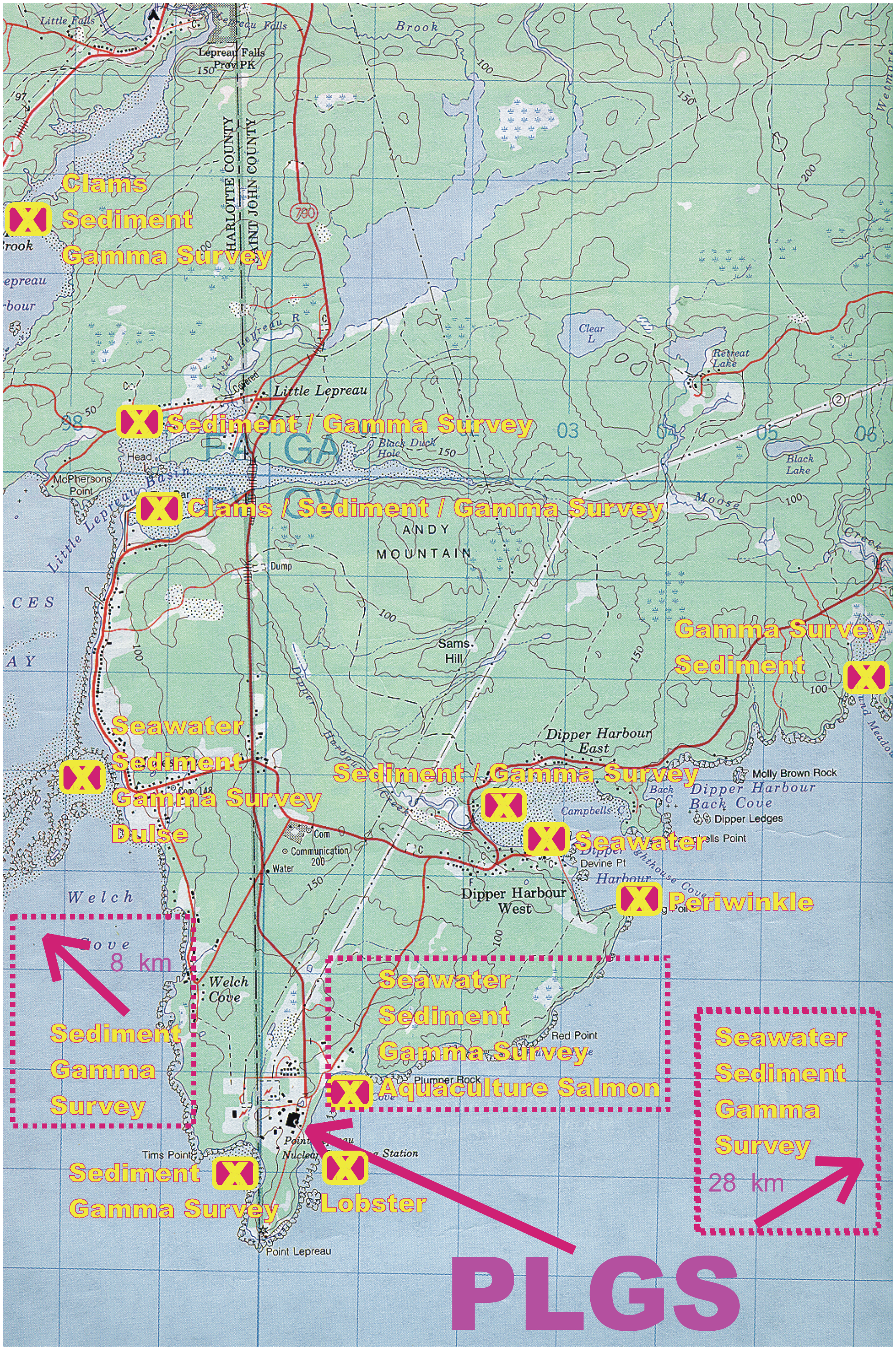


Figure 3.05: Marine Monitoring Sites

4 Summary and Discussion of REMP Data

The following is a summary and discussion of the data on environmental samples collected for the year 2015.

Most samples contained low levels of naturally occurring K-40 or cosmogenically produced Be-7. Some samples contained Cs-137 (soils, sediments, lichen) from the atmospheric weapons tests of past years and international events (at Chernobyl and Fukushima). One soil sample near the SRWMF showed an elevated Cs-137 activity. Tritium (in air and fresh water) is the only radionuclide originating from PLNGS that is detected consistently. In 2015, analyses that indicated emissions traceable to PLNGS were:

- H-3 in airborne water vapour and fresh water
- H-3 in Parshall flume and bore hole water from the Solid Radioactive Waste Management Facility (SRWMF)
- H-3 in water from onsite monitoring wells

The only assessable radiation dose from PLNGS on the local population is that from tritiated water vapour in air. Offsite, the activity of H-3 in air ranges from less than 2E-02 $Bq \cdot m^{-3}$ (below the lower limit of detection by the method used) to approximately 2E+00 $Bq \cdot m^{-3}$ of air. The natural concentration of H-3 is up to 7E-01 $Bq \cdot L^{-1}$ in most surface waters and up to 1E-03 $Bq \cdot m^{-3}$ in air.

The natural concentration of C-14 in the atmosphere is approximately 4E-2 $Bq \cdot m^{-3}$. This level is usually detected by the sensitive analytical method used in the monitoring program.

Only detected radionuclides are listed in the following tables. (Refer to Tables A.01 to A.11 in Appendix A for detailed listings of detection limits. Refer to Appendix C for a listing of location codes.) Most tables contain the following data:

Column 1 - Shows the type of analysis or nuclide.

Column 2 - Shows the total number of samples analysed.

Column 3 – Shows the mean of the Critical Levels (CLs) for all samples. Any measurement greater than the CL is considered detected at the 99% confidence level (an explanation of the statistical protocol is given in Appendix A).

Column 4 - Shows the range of the Critical Levels (CLs) for all samples. Any measurement greater than the CL is considered detected at the 99% confidence level (an explanation of the statistical protocol is given in Appendix A).

Column 5 - Shows the mean of the detected values (i.e., values exceeding the CL) for all Indicator stations.

Column 6 - Shows the ratio of the number of detected values to the total number of Indicator samples.

Column 7 - Shows the range of detected values for the Indicator stations.

Column 8 - Shows the mean of detected values at the Reference location(s).

Column 9 - Shows the ratio of detected values to the total number of samples at this location.

Column 10 - Shows the range of detected values for the Reference location(s).

4.01 Airborne Particulates

Gross alpha was detected on 88 filters analysed, gross beta on 89, Be-7 on 79 and K-40 on one. None of these results are attributable to the operation of PLNGS.

Air is continuously monitored from the eight locations shown in Figure 3.02. Once a month the filters are changed and analysed. In early 2015, sampling at the reference location (A01R) was terminated due to safety and security concerns for personnel and equipment. A new location (A13R) became operational late in 2015.

Gross alpha and gross beta measurements are an indication of total activity in the environment. This includes naturally occurring radon progeny, cosmogenic (Be-7), and person-made sources of radiation. The maximum concentration of gross beta in air onsite was 1.4E-03 $Bq{\cdot}m^{-3}$ of air. Offsite gross beta reached 7.5E-04 $Bq{\cdot}m^{-3}$.

When Sr-89,90 emissions are low, the expected concentration of these radionuclides in environmental air samples is below the detection limit. The GEM monitors PLNGS gaseous emissions continuously at their source. The GEM filter was changed daily and the weekly composite is sent to the lab for analysis until August 2015 when a new GEM was put in service. After that the filter was changed weekly. Fifty-two sets of GEM filters were analysed for Sr-89,90. If the weekly emission is more than one percent of the weekly DRL, or if elevated beta activity is detected in environmental air samples, a Sr-89,90 analysis is performed on the environmental air particulate samples. Since no Sr-89 or Sr-90 emissions were detected in 2015, no further analyses were required.

Table 4.01 is a summary of detected radionuclides. Figures 4.01 and 4.02 show the gross beta results for each location throughout the year.

Table 4.01: Airborne Particulates ($Bq \cdot m^{-3}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
ALPHA	89	2.8E-6	1.1E-6 to 1.2E-5	2.2E-5	83/84	3.1E-6 to 1.2E-4	3.8E-5	5/5	1.5E-5 to 5.9E-5
BETA	89	6.8E-6	2.7E-6 to 2.9E-5	3.0E-4	84/84	3.0E-5 to 1.4E-3	4.0E-4	5/5	1.8E-4 to 7.5E-4
Be-7	89	1.2E-4	3.4E-5 to 6.3E-4	1.4E-3	74/84	1.3E-4 to 3.7E-3	1.3E-3	5/5	3.2E-4 to 2.5E-3
K-40	89	3.6E-4	1.4E-4 to 1.5E-3	3.3E-4	1/84	3.3E-4 to 3.3E-4	*	*	*

*The activity is less than or equal to the Critical Level (99% Confidence Level).

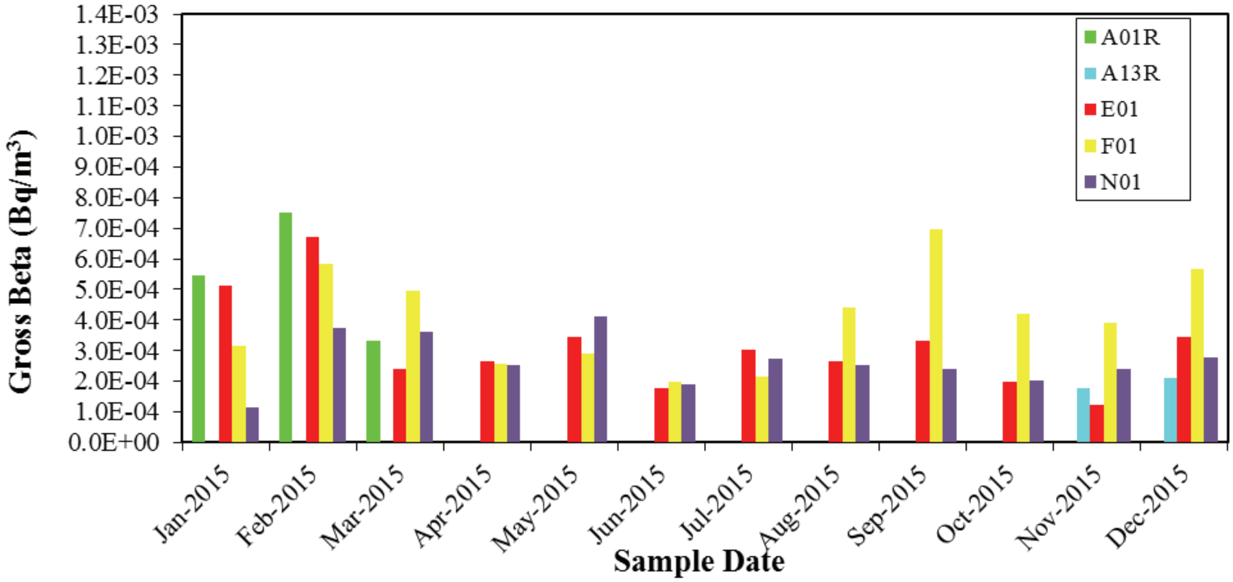


Figure 4.01: Gross Beta (Air Particulates) at Offsite Air Stations

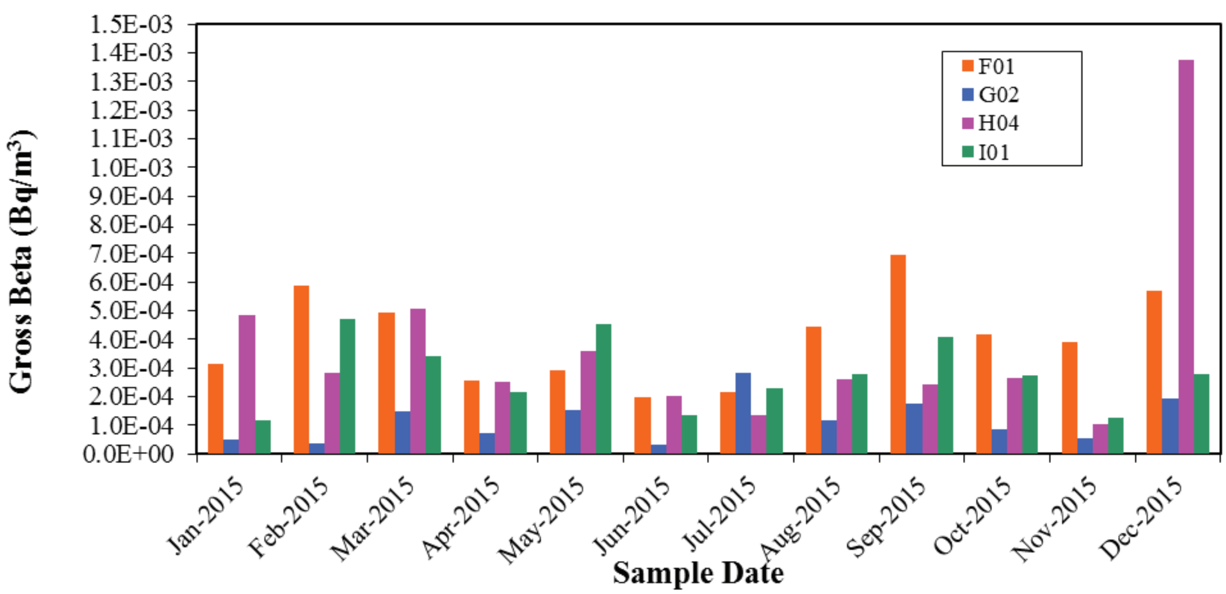


Figure 4.02: Gross Beta (Air Particulates) at Onsite Air Stations

4.02 Airborne Iodines

No radioiodines were detected in any of the 89 samples analysed.

Air is monitored continuously, using charcoal cartridges, from the eight locations shown in Figure 3.02. Once a month the cartridges are changed and analysed. In early 2015, sampling at the reference location (A01R) was terminated. A new location (A13R) became operational late in 2015.

Iodine-131 was consistently below the Critical Level (average 1.1E-05 $Bq{\cdot}m^{-3}$).

4.03 Water Vapour

Tritium was detected in 71 of the 82 samples collected from the air monitoring stations on the Point Lepreau peninsula, and in none of the four samples from the reference location.

Water vapour is collected continuously in molecular sieve bombs from the eight locations shown in Figure 3.02. Once a month the bombs are changed and analysed. In early 2015, sampling at the reference location (A01R) was terminated. A new location (A13R) became operational late in 2015.

The maximum concentration of tritium in air onsite was 8.5E+00 $Bq{\cdot}m^{-3}$ of air. Offsite it reached 1.6E+00 $Bq{\cdot}m^{-3}$. Tritium has been detected occasionally at the reference location, even before PLNGS became operational. In early 2015, sampling at the reference location (A01R) was terminated due to safety and security concerns. A new location (A13R) became operational late in 2015.

Table 4.02 is a summary of the tritium data and Table 4.03 gives details of the tritium results by location. Figures 4.03 and 4.04 show the H-3 results for each location. “Less Than” values are plotted for non-detected results. Generally, locations to the northeast (H04, I01 and N01) have elevated H-3 measurements in the warmer months due to the predominant summer wind direction which influences where the H-3 is detected. This changes in the winter to impact the southwest locations (G02 and L04).

When H-3 emissions are low, the expected H-3 concentration in other environmental samples is below the detection limit. If the weekly H-3 emissions are more than one percent of the weekly DRL, a H-3 analysis is performed on berries and garden vegetables. Since the H-3 emissions in 2015 were 5E-02% DRL (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.05 shows the weekly H-3 emissions from PLNGS. “Less Than” values are plotted for non-detected results.

Table 4.02: Water Vapour (Bq·m^3)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	86	7.7E-2	1.9E-2 to 1.5E-1	1.3E+0	71/82	2.3E-2 to 8.5E+0	*	*	*

*The activity is less than or equal to the Critical Level (99% Confidence Level).

Table 4.03: Tritium (Water Vapour) at Each Air Station (Bq·m^{-3})

Location Code		***A01R/A13R***	***E01***	***F01***	***G02***	***H04***	***I01***	***L04***	***N01***
Location		***Bocabec / Saint Andrews***	***Maces Bay***	***Welch Cove***	***Lepreau Lighthouse***	***Former Information Centre Site***	***SRWMF***	***Construction Stores***	***Dipper Harbour***
Distance from PLNGS		***45 km***	***4.5 km***	***1.6 km***	***1.0 km***	***0.75 km***	***1.2 km***	***0.55 km***	***3.7 km***
Collection Start Date The sample collection periods are approximately one month in duration. All sample stations are changed at the same time. The start date is the stop date for the previous sample.	2015-01-07	<4.3E-2*	<3.9E-2	6.9E-2	3.7E-1	2.3E-1	7.3E-1	2.6E-1	N/A
	2015-02-11	<4.3E-2*	<4.4E-2	4.7E-2	2.3E-1	6.3E-1	5.4E-1	2.07E-1	<3.5E-2
	2015-03-10	1.7E-2*	1.8E-2	4.7E-2	2.3E-2	6.2E-1	5.4 E-1	2.0E-1	1.6E-2
	2015-04-08	terminated	1.4E-1	1.6E+0	7.0E +0	7.18E+0	8.5E+0	6.4E+0	N/A
	2015-05-06	terminated	<1.1E-1	1.7E-1	3.2E-0	<1.6E-1	2.0E+0	1.0E-0	7.67E-1
	2015-06-03	terminated	<1.9E-1	1.8E-1	2.8E-1	1.6E+0	1.7E+0	6.7E-1	1.7E-1
	2015-07-02	terminated	1.4E-1	6.1E-1	<1.6E-1	1.2E+0	2.0E+0	3.3E-1	1.0E+0
	2015-08-05	terminated	1.9E-1	1.03E+0	<1.45E-1	3.1E+0	2.9E+0	2.7E+0	1.5E+0
	2015-09-02	terminated	<2.3E-1	1.8E-1	4.8E-1	1.3E+0	2.7E+0	3.6E-1	3.9E-1
	2015-10-01	terminated	<1.3E-1	2.2E-1	1.7E+0	2.7E+0	3.4E+0	6.7E-1	4.4E-1
	2015-11-05	<1.5E-1**	1.0E-1	1.3E+0	4.0E-1	1.4E+0	1.1E+0	4.6E-1	1.3E-1
	2015-12-04	<7.7E-2**	1.5E-1	N/A	1.2E+0	4.2E-1	8.3E-1	9.9E-1	1.5E-1

*A01R **A13R (in service on 2015-11-12)
NA: Data not available due to equipment failure.

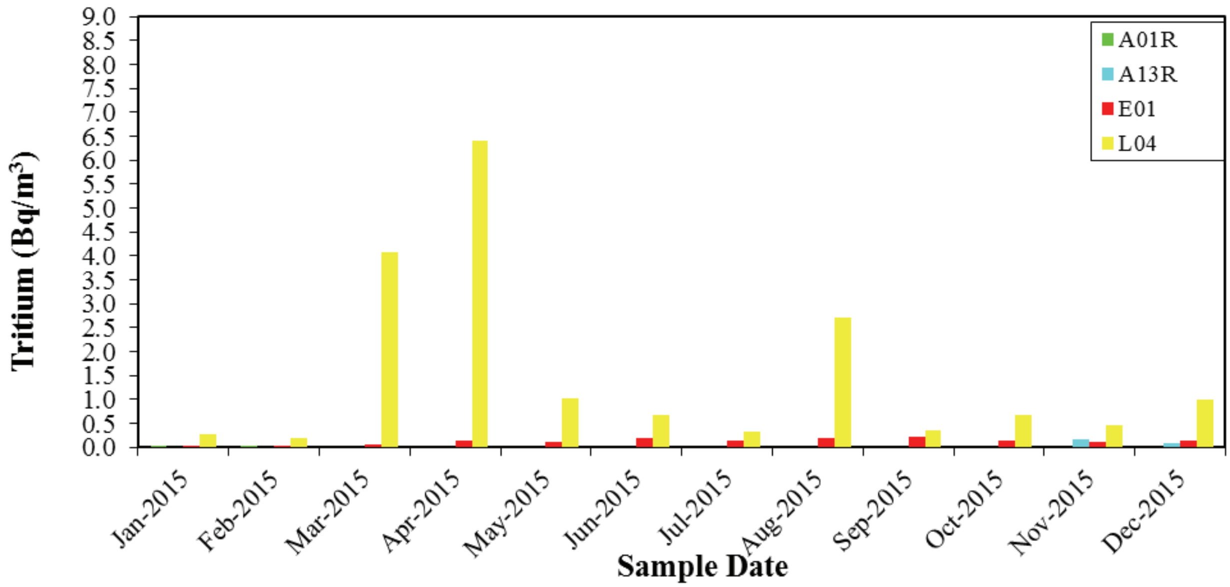


Figure 4.03: Tritium (Water Vapour) at Offsite Air Stations

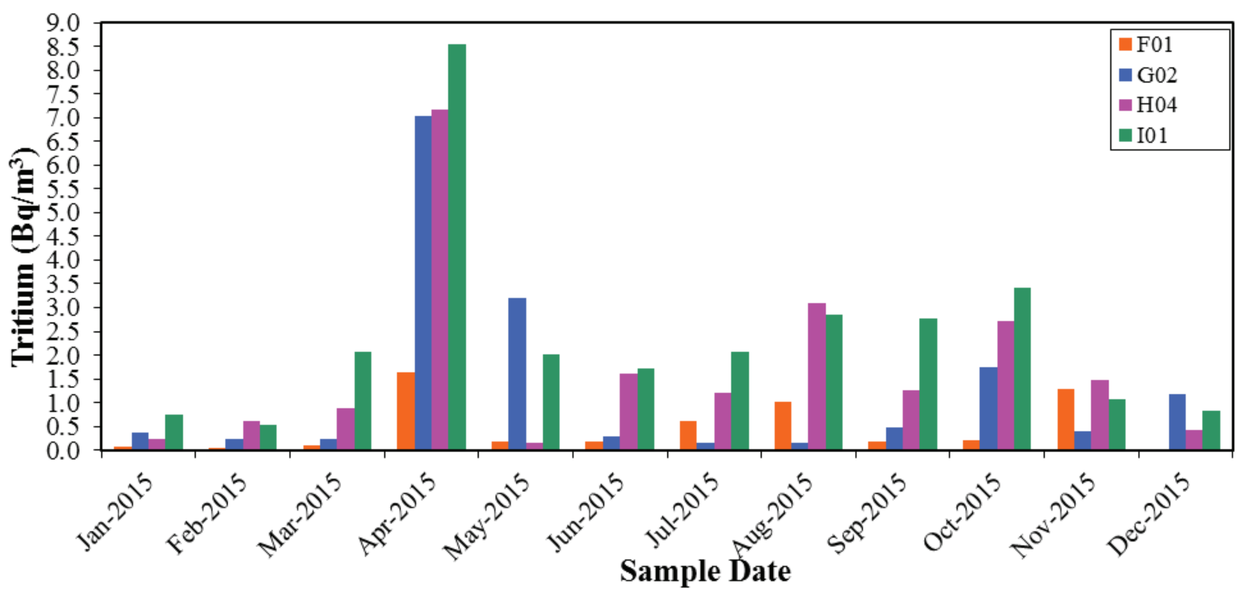
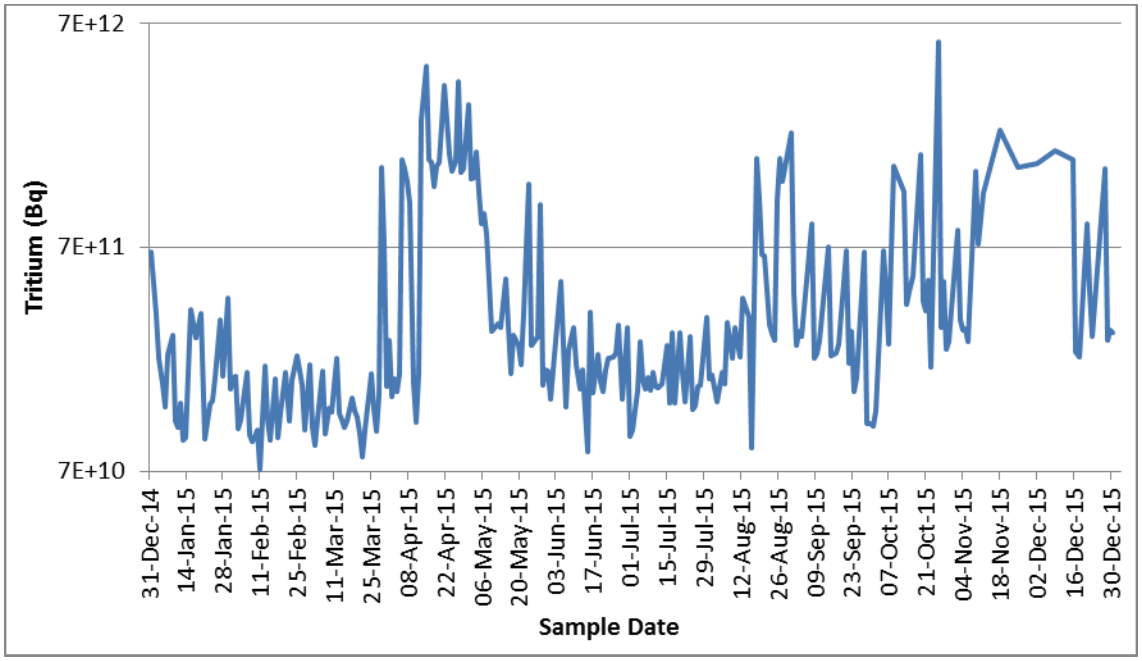


Figure 4.04: Tritium (Water Vapour) at Onsite Air Stations



Note: The Weekly DRL for H-3 is 5.4E+15 Bq

Figure 4.05: Gaseous H-3 Emissions for 2015

4.04 Carbon Dioxide

Carbon-14 was detected in 22 of the 23 samples from the onsite monitors and nine of the 12 samples from the offsite monitor.

Air is continuously bubbled through a caustic solution at two onsite locations (G02 and H04 in Figure 3.02) and at one reference location. The caustic bubblers are changed monthly and returned to the lab for analysis.

The maximum concentration of gaseous C-14 onsite was 2.0E-01 Bq·m^{-3}. Offsite the gaseous C-14 concentration reached 7.6E-02 Bq·m^{-3}. Based on stack emissions, the calculated incremental concentration of C-14 in air at the boundary fence for 2015 was less than 4E-04 Bq·m^{-3} (a small fraction of the natural level of 4E-02 Bq·m^{-3}).

A summary of the analysis results is given in Table 4.04. Table 4.05 gives details of C-14 results.

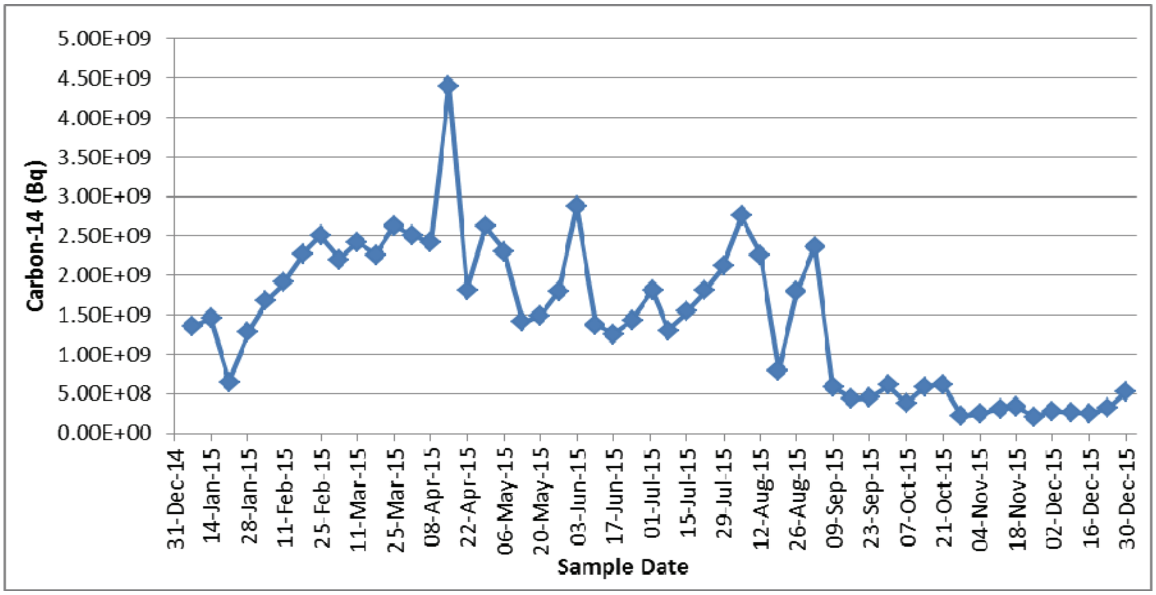
When C-14 emissions are low, the expected concentration of C-14 in other environmental samples is below the detection limit. If the weekly C-14 emission is more than one percent of the weekly DRL, a C-14 analysis is performed on berries, milk, water and garden vegetables. Since the C-14 emissions in 2015 were 1E-03% DRL (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.06 shows the weekly C-14 emissions from PLNGS. "Less Than" values are plotted for non-detected results.

Table 4.04: Carbon Dioxide ($Bq \cdot m^{-3}$)

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
C-14	35	2.7E-2	2.1E-2 to 6.1E-2	6.0E-2	22/23	3.4E-2 to 2.0E-1	5.9E-2	9/12	3.6E-2 to 7.6E-2

Table 4.05: Carbon-14 (Carbon Dioxide) at Each Monitoring Location (Bq·m^{-3})

Location Code		***G02***	***H04***	***X03R***
Location		***Lepreau Lighthouse***	***Former Information Centre Site***	***Fredericton Laboratory***
Distance from PLNGS		***1.0 km***	***0.75 km***	***100 km***
Collection Start Date The sample collection periods are approximately one month in duration. All sample stations are changed at the same time. The start date is the stop date for the previous sample.	2015-01-07	6.5E-2	6.9E-2	<6.2E-2
	2015-02-11	6.3E-2	4.7E-2	<4.1E-2
	2015-03-10	3.4E-2	4.21E-2	<5.5E-2
	2015-04-08	<1.02E+0	3.6E-2	5.6E-2
	2015-05-06	5.7E-2	4.0E-2	6.1E-2
	2015-06-03	5.3E-2	5.6E-2	6.0E-2
	2015-07-02	5.4E-2	5.1E-2	7.6E-2
	2015-08-05	4.8E-2	4.7E-2	7.29E-2
	2015-09-02	<4.00E-2	5.7E-2	4.5E-2
	2015-10-01	5.7E-2	4.9E-2	5.9E-2
	2015-11-05	4.8E-2	4.8E-2	6.2E-2
	2015-12-04	2.0E-1	1.04E-1	3.6E-2



Note: The Weekly DRL for C-14 is 1.3E+14 Bq

Figure 4.06: Gaseous C-14 Emissions for 2015

4.05 Ambient Gamma Measurements (TLD)

Gamma exposure measurements were slightly elevated onsite compared with offsite. The elevated measurements were at the locations near the SRWMF and reactor building.

Ambient gamma radiation is measured by TLDs at the 76 locations shown in Figure 3.04. Forty-six of these locations are near the SRWMF. TLDs are changed quarterly. Ten of the 304 dosimeters placed in the environment were lost or damaged (due to severe winter). A new method of attaching the TLDs to structures has since been implemented to mitigate this in the futute Three of the dosimeters were not placed because the reference location (A01R) was terminated due to safety and security concerns. A new location (A13R) became operational late in 2015.

The average measurement at the SRWMF (890 $\mu Gy \cdot a^{-1}$) is higher than for other onsite locations (656 $\mu Gy \cdot a^{-1}$) and boundary locations (646 $\mu Gy \cdot a^{-1}$). The measurements at other onsite locations are not significantly different from those at offsite locations (683 $\mu Gy \cdot a^{-1}$) and that at the reference location (506 $\mu Gy \cdot a^{-1}$). A new location was added in 2001 in a community (York Mills) 120 km north west of PLNGS. The area is noted for its natural uranium content and the measurement at this site (1364 $\mu Gy \cdot a^{-1}$) is higher than the highest location at PLNGS.

Data are given in Table 4.06. Increases in measurements at the SRWMF locations (I11A to I11T on the perimeter fence of the SRWMF-Phase 1, I21A to I21L on the perimeter fence of the SRWMF-Phase 2 and I31A to I31T on the perimeter fence of the SRWMF-Phase 3) are due to low-level waste, used fuel emplacement and refurbishment waste, and not to station emissions. There were 182 concrete canisters filled to the end of 2015 with 98 460 used-fuel bundles. A small, but indefinable, portion of the measurement on the TLDs at the SRWMF is due to enhanced natural radiation from the aggregate used in making the concrete structures. Figure 4.07 compares the reference location results with the results for other locations.

Table 4.06: Ambient Gamma – TLD (µGy)

Location	Dose (µGy± 10%)				
	1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Year
A01R	127 ± 13	Terminated	Terminated	Terminated	510 ± 10
B03	139 ± 14	165 ± 16	192 ± 19	175 ± 18	670 ± 30
C03	168 ± 17	189 ± 19	235 ± 24	214 ± 21	810 ± 40
D02	130 ± 13	166 ± 17	209 ± 21	177 ± 18	680 ± 30
E01	168 ± 17	142 ± 14	169 ± 17	142 ± 14	620 ± 30
E04	126 ± 13	168 ± 17	205 ± 21	179 ± 18	680 ± 30
E05	132 ± 13	150 ± 15	188 ± 19	162 ± 16	630 ± 30
E06	254 ± 25	239 ± 24	254 ± 25	219 ± 22	970 ± 50
F01	125 ± 13	118 ± 12	136 ± 14	131 ± 13	510 ± 30
G02	168 ± 17	181 ± 18	196 ± 20	184 ± 18	730 ± 40
H04	140 ± 14	143 ± 14	152 ± 15	146 ± 15	580 ± 30
H05	99 ± 10	105 ± 10	133 ± 13	114 ± 11	450 ± 20
I11A	168 ± 17	212 ± 21	241 ± 24	222 ± 22	840 ± 40
I11B	174 ± 17	218 ± 22	255 ± 25	225 ± 22	870 ± 40
I11C	182 ± 18	204 ± 20	236 ± 24	221 ± 22	840 ± 40
I11D	157 ± 16	208 ± 21	241 ± 24	211 ± 21	820 ± 40
I11E	164 ± 16	201 ± 20	239 ± 24	224 ± 22	830 ± 40
I11F	168 ± 17	224 ± 22	305 ± 31	285 ± 29	980 ± 50
I11J	179 ± 18	238 ± 24	257 ± 26	259 ± 26	930 ± 50
I11K	149±15	210 ± 21	245 ± 24	215 ± 21	820 ± 40
I11L	141 ± 14	208 ± 21	223 ± 23	206 ± 21	790 ± 40
I11M	185±18	249 ± 25	343± 34	335 ± 33	1110 ± 60
I11N	N\A	203 ± 20	276 ± 28	227±23	940 ± 40
I11O	160 ± 16	221 ± 22	253 ± 25	N\A	850 ± 40
I11P	162 ± 16	215 ± 21	274 ± 27	224 ± 22	880 ± 40
I11Q	169 ± 17	230 ± 23	272 ± 27	230 ± 23	900 ± 50
I11S	157 ± 16	213 ± 21	259 ± 26	N/A	840 ± 40
I11T	178 ± 18	255 ± 25	281 ± 28	273 ± 27	990 ± 50
I21A	174 ± 17	186 ± 19	237 ± 24	194 ± 19	790 ± 40
I21B	212 ± 21	239 ± 24	266 ± 27	208 ± 21	920 ± 50
I21C	185 ± 19	197 ± 20	222 ± 22	N/A	810 ± 30
I21D	N/A	262 ± 26	299 ± 30	260 ± 26	1090 ±50
I21E	210± 21	259 ± 26	287 ± 29	246 ± 25	1000 ± 50
I21F	219± 22	232 ± 23	214 ± 21	200 ± 20	860 ± 40
I21G	187± 19	190 ± 19	207 ± 21	193 ± 19	780 ± 40
I21H	178± 18	210 ± 21	234 ± 23	214 ± 21	840 ± 40
I21I	183± 18	212 ± 21	230 ± 23	201 ± 20	830 ± 40
I21J	204± 20	229 ± 23	251 ± 25	245 ± 24	930 ± 50
I21K	188± 19	192 ± 19	242 ± 24	207 ± 21	830 ± 40
I21L	192± 19	203 ± 20	244 ± 24	204 ± 20	840 ± 40
I31A	140± 14	191 ± 19	223 ± 22	198 ± 20	750 ± 40

Table 4.06 (continued): Ambient Gamma – TLD (µGy)

Location	Dose (µGy± 10%)				
	1st Quarter	2nd Quarter	3rd Quarter	4th Quarter	Year
I31B	166 ± 17	211 ± 21	235 ± 23	209 ± 21	820 ± 40
I31C	163 ± 16	227 ± 23	280 ± 28	222 ± 22	890 ± 50
I31D	180 ± 18	244 ± 24	262 ± 26	248 ± 25	930 ± 50
I31E	181± 18	227±23	266 ± 27	236 ± 24	910 ± 50
I31F	N/A	277 ± 28	263 ± 26	262 ± 26	1070 ± 50
I31G	194 ± 19	241 ± 24	275± 28	258 ± 26	970 ± 50
I31H	192 ± 19	241 ± 24	273 ± 28	239 ± 24	940 ± 50
I31I	166 ± 17	223 ± 22	264 ± 26	N/A	870 ± 40
I31J	152 ± 15	228 ± 23	256 ± 26	243 ± 24	880 ± 40
I31K	198 ± 20	220 ± 22	261 ± 26	228 ± 23	910 ± 50
I31L	N/A	210±21	244 ± 24	213 ± 21	890 ± 40
I31M	191± 19	227 ± 23	246 ± 25	220 ± 22	880 ± 40
I31N	208± 21	221 ± 22	239 ± 24	222±22	890 ± 40
I31P	225± 22	230 ± 23	249 ± 25	238 ± 24	940 ± 50
I31Q	205± 20	234 ± 23	242 ± 24	234 ± 23	910 ± 50
I31S	266± 27	243 ± 24	249 ± 25	227± 23	990 ± 50
I31T	154± 15	204 ± 20	230 ± 23	210± 21	700 ± 40
I86	134± 13	143± 14	181 ± 18	152 ± 15	610 ± 30
I87	131± 13	148± 15	172 ± 17	160 ± 16	610 ± 30
I88	131± 13	160± 16	177 ± 18	191 ± 19	660 ± 30
I89	127± 13	164 ± 16	182 ± 18	163 ± 16	640 ± 30
J20	157± 16	189± 19	206 ± 21	N/A	740 ± 30
J35	198± 20	191± 19	198 ± 20	182 ± 18	770 ± 40
K01	214± 21	172 ± 17	191 ± 19	174 ± 17	750 ± 40
L01	183± 18	187 ± 19	215 ± 21	204 ± 20	790 ± 40
L03	159± 16	134 ± 13	160 ± 16	N/A	600 ± 30
L04	168± 17	166 ± 17	193 ± 19	170 ± 17	600 ± 30
M02	141± 14	127 ± 13	139 ± 14	131 ± 13	540 ± 30
N01	145± 15	155 ± 15	177 ± 18	155 ± 16	630 ± 30
P03	112± 11	152± 15	177 ± 18	189 ± 19	630 ± 30
X12	350± 35	335 ± 33	367 ± 37	312 ± 31	1360 ± 70
YTL1	133± 13	128 ± 13	138 ± 14	115 ± 12	510 ± 30
YTL2	145± 15	134 ± 13	146 ± 15	134 ± 13	560 ± 30
YTL3	120± 12	111 ± 11	133 ± 13	112 ± 11	480 ± 20
YTL4	114± 11	107 ± 11	135 ± 14	109 ± 11	470 ± 20

NA: Data Not Available – Lost or damaged TLD for this collection interval.

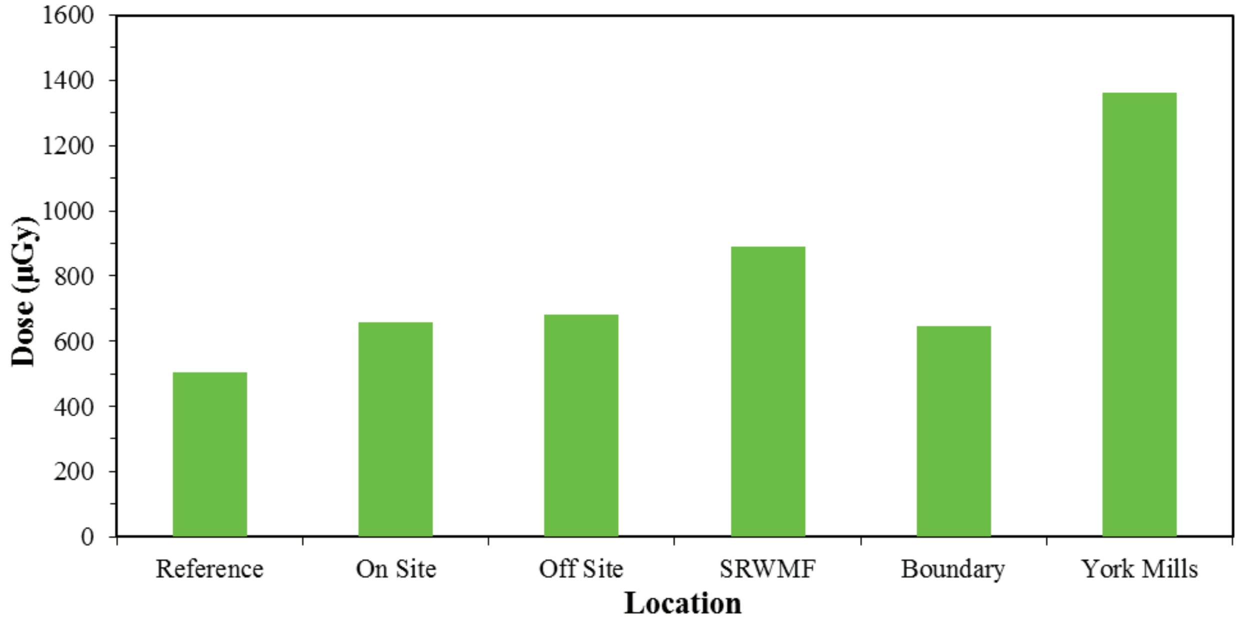


Figure 4.07: Mean Ambient Gamma (TLD) Results

4.06 Milk

Of the 24 samples analysed, K-40 was detected in 24, Cs-137 in one and H-3 in two. None of these results are attributable to the operation of PLNGS (the H-3 results were deemed false positives).

There are no commercial herds or individual cows producing milk in the Lepreau area. The closest herds to PLNGS are in Lynnfield (70 km to the northwest), Fredericton Junction (70 km to the north), and Hammond River (60 km to the northeast). Milk from these locations is analysed quarterly. Milk from a commercial dairy is purchased each month from a supermarket in Fredericton. All milk samples are analysed for gamma emitting radionuclides and tritium.

Since C-14 emissions are low, the expected concentration of C-14 in milk is below the detection limit. If the weekly C-14 emissions are more than one percent of the weekly DRL, a C-14 analysis is performed on milk. Since the C-14 emissions in 2015 were 1E-03% DRL (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.06 shows the weekly C-14 emissions.

Cesium-137 was detected (4.9 E-02 $Bq \cdot L^{-1}$) in one sample. It is occasionally detected in milk. The Cs-137 originates from fallout of weapons tests and international events (Chernobyl & Fukushima), and concentrations have been declining ever since. Milk samples analysed between 1977 and 1982, before PLNGS began operations, contained an average Cs-137 concentration of 1.2E+00 $Bq \cdot L^{-1}$. Cows preferentially absorb cesium if there is a dietary deficiency in potassium.

Naturally occurring K-40 (average of 4.6E+01 $Bq \cdot L^{-1}$) was also detected in milk.

PLNGS emissions of tritium and gamma emitters were too low throughout the year to be detected in these milk samples. The two detected tritiums were deemed false positives.

Table 4.07 is a summary of the detected radionuclides in milk.

4.07 GEM Particulates (Sr-89,90)

When Sr-89,90 emissions are low, the expected concentration of Sr-89,90 in environmental air samples is below the detection limit. The GEM monitors PLNGS gaseous emissions continuously at their source. The GEM filter is changed daily and the weekly composite is sent to the Fredericton lab for analysis. Fifty-two sets of these GEM filters were analysed for Sr-89,90. If the weekly emissions are more than one percent of the weekly DRL, or if elevated beta activity is detected in environmental air samples, a Sr-89,90 analysis is performed on these environmental air samples. Since no Sr-89 or Sr-90 emissions were detected in 2015, no further analyses were required.

Table 4.07: Milk ($Bq \cdot L^{-1}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	24	1.3E+1	1.1E+1 to 1.4E+1	2.8E+1	2/12	1.5E+1 to 4.2E+1	*	*	*
K-40	24	4.6E-1	3.0E-1 to 6.3E-1	5.6E+1	12/12	5.1E+1 to 6.0E+1	5.8E+1	12/12	5.3E+1 to 6.0E+1
Cs-137	24	5.4E-2	4.5E-2 to 6.2E-2	4.9E-2	1/12	4.9E-2 to 4.9E-2	*	*	*

*The activity is less than or equal to the Critical Level (99% Confidence Level).

4.08 Well Water

Of the 25 samples analysed, gross alpha was detected in five, gross beta was detected in 10 and H-3 in seven. Only the H-3 results are attributable to the operation of PLNGS.

Water is collected semi-annually from the 11 locations shown in Figure 3.03. Two of these wells are onsite. Ten additional wells are sampled once per year. These wells are located just outside the exclusion boundary and belong to local residents.

The alpha (up to 1.6E+00 $Bq{\cdot}L^{-1}$) and beta (up to 5.2E-01 $Bq{\cdot}L^{-1}$) activities are due to the presence of naturally occurring radionuclides particular to certain locations. Detected H-3 concentrations ranged from 1.2E+01 to 2.4E+01 $Bq{\cdot}L^{-1}$. Tritium from PLNGS emissions washes out into precipitation and subsequently drains into some of the wells. Precipitation analyses (Section 4.14) indicate H-3 concentrations ranging from 1.7E+01 to 4.3E+02 $Bq{\cdot}L^{-1}$ in 20 of 29 samples.

Since C-14 emissions are low, the expected concentration of C-14 in well water is below the detection limit. If the weekly C-14 emissions are more than one percent of the weekly DRL, a C-14 analysis is performed on well water. Since the C-14 emissions in 2015 were 1E-03% DRL (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.08 shows the weekly C-14 emissions.

Table 4.08 is a summary of the detected radionuclides in well water. The H-3 measurements were made after samples had been allowed to sit for up to two weeks to reduce radioactive interference from the relatively abundant, but short half-life, radon progeny which are common in most well waters.

The Health Canada, 2010 *Guidelines for Canadian Drinking Water Quality* (Federal-Provincial-Territorial Committee on Drinking Water of the Federal-Provincial-Territorial Committee on Health and the Environment) recommends 7.0E+03 $Bq{\cdot}L^{-1}$ as the maximum acceptable average concentration for H-3 in drinking water.

Table 4.08: Well Water (Bq·L^{-1})

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
ALPHA	25	9.1E-2	3.1E-2 to 1.6E-1	5.2E-1	5/25	1.6E-1 to 1.6E+0	*	*	*
BETA	25	5.4E-2	3.0E-2 to 6.9E-2	1.8E-1	10/25	5.8E-2 to 5.2E-1	*	*	*
H-3	25	1.3E+1	1.1E+1 to 1.3E+1	1.8E+1	7/25	1.2E+1 to 2.4E+1	*	*	*

*The activity is less than or equal to the Critical Level (99% Confidence Level).

4.09 Pond/Puddle/Surface Water

Low levels of H-3 were detected in 17 of the 34 samples. No gamma emitters were detected in these samples.

This category includes ponds, lakes, streams and runoff samples. Most of these samples are from onsite locations. Two important offsite locations, sampled quarterly, are the freshwater supply reservoirs for Saint John and PLNGS, at Spruce Lake and Hanson Stream, respectively.

Detected H-3 activities ranged from 1.9E+01 to 8.9E+01 $Bq \cdot L^{-1}$. Variability can be due to the size of the water reservoir and the length of time the sample has remained at the location. Tritium from PLNGS emissions washes out into precipitation. Precipitation analyses (Section 4.14) indicate H-3 concentrations ranging from 1.7E+01 to 4.3E+02 $Bq \cdot L^{-1}$ in 20 of 29 samples.

Since C-14 emissions are low, the expected concentration of C-14 in water is below the detection limit. If the weekly C-14 emissions are more than one percent of the weekly DRL, a C-14 analysis is performed on water. Since the C-14 emissions in 2015 were 1E-03% DRL (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.08 shows the weekly C-14 emissions.

Table 4.09 is a summary of the detected radionuclides in surface water.

Table 4.09: Pond/Puddle/Surface Water (Bq·L^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	34	1.3E+1	1.2E+1 to 1.5E+1	4.8E+1	17/34	1.9E+1 to 8.9E+1	*	*	*

*There is no reference location.

4.10 Berries

No radionuclides were detected in the three samples analysed.

Berries are sampled weekly when in season. Three samples of blueberries were collected from Pennfield.

It is not unusual to detect Cs-137 in berries throughout North America (from fallout of past atmospheric weapons tests), although none was detected this year.

Since H-3 and C-14 emissions are low, the expected concentrations of H-3 and C-14 in berries are below the detection limits. If the H-3 or C-14 weekly emissions are more than one percent of the weekly DRL, then H-3 or C-14 analysis is performed on berries. Since the emissions in 2015 were 5E-02% DRL for H-3 and 1E-03% DRL for C-14 (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.05 shows the weekly H-3 emissions and Figure 4.06 shows the weekly C-14 emissions.

4.11 Garden Vegetables

Potassium-40 was detected in 16 of the 20 samples analysed. These results are not attributable to the operation of PLNGS.

All samples were taken from a local garden in Dipper Harbour (4 km from PLNGS in the predominant downwind direction). These samples were supplied weekly during the growing season.

As in most food samples, naturally occurring K-40 was detected in 16 of the 20 samples (6.3E+01 to 7.2E+02 $Bq\cdot kg^{-1}$).

Since H-3 and C-14 emissions are low, the expected concentrations of H-3 and C-14 in garden vegetables are below the detection limit. If the H-3 or C-14 weekly emissions are more than one percent weekly DRL, then H-3 or C-14 analysis is performed on garden vegetables. Since the emissions in 2015 were 5E-02% DRL for H-3 and 1E-03% DRL for C-14 (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.05 shows the weekly H-3 emissions and Figure 4.06 shows the weekly C-14 emissions.

Table 4.10 is a summary of the detected radionuclides in garden vegetables.

4.12 Vegetation (Lichen)

Of the 13 samples analysed, Be-7 was detected in nine. These results are not attributable to the operation of PLNGS.

These samples are collected whenever and wherever available from onsite locations.

Different species of lichen and moss concentrate a wide range of radionuclides and are sensitive indicators of radionuclides in the environment. Cosmogenically produced Be-7 was detected in nine samples (2.8E+02 to 5.2E+02 $Bq\cdot kg^{-1}$).

Table 4.11 is a summary of the detected radionuclides in vegetation.

Table 4.10: Garden Vegetables ($Bq \cdot kg^{-1}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	20	5.3E+1	1.0E+1 to 2.4E+2	1.9E+2	16/20	6.3E+1 to 7.2E+2	*	*	*

*There is no reference location.

Table 4.11: Vegetation ($Bq \cdot kg^{-1}$)

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
Be-7	13	1.5E+2	2.4E+1 to 3.9E+2	3.7E+2	9/13	2.8E+2 to 5.2E+2	*	*	*

*There is no reference location.

4.13 Soil

Of the 28 samples analysed, Cs-137 was detected in 19, Ac-228 in 19 and K-40 in 27. One Cs-137 result is attributable to the operation of PLNGS.

Soil samples are taken quarterly from the eight air monitoring location sites shown in Figure 3.02 and from the local elementary school. Samples are collected at E02 rather than E01 due to a lack of readily available soil at that site.

Twenty-seven samples contained naturally occurring K-40 (3.4E+02 to 8.2E+02 $Bq \cdot kg^{-1}$), 19 samples contained naturally occurring Ac-228 (1.6E+01 to 5.5E+01 $Bq \cdot kg^{-1}$) and 19 samples contained Cs-137 (9.4E-01 to 9.4E+01 $Bq \cdot kg^{-1}$) . All Cs-137 results were at typical levels for the region except the one that was collected near the SRWMF. Cesium-137 from fallout of past atmospheric weapons tests and reactor accidents tends to accumulate in the organic layer of the soil. Most fluctuation in Cs-137 and K-40 levels seems to be due to the quantity of organic load in the sample.

Table 4.12 is a summary of the detected radionuclides in soil. Figure 4.08 shows individual Cs-137. "Less Than" values are plotted for non-detected results.

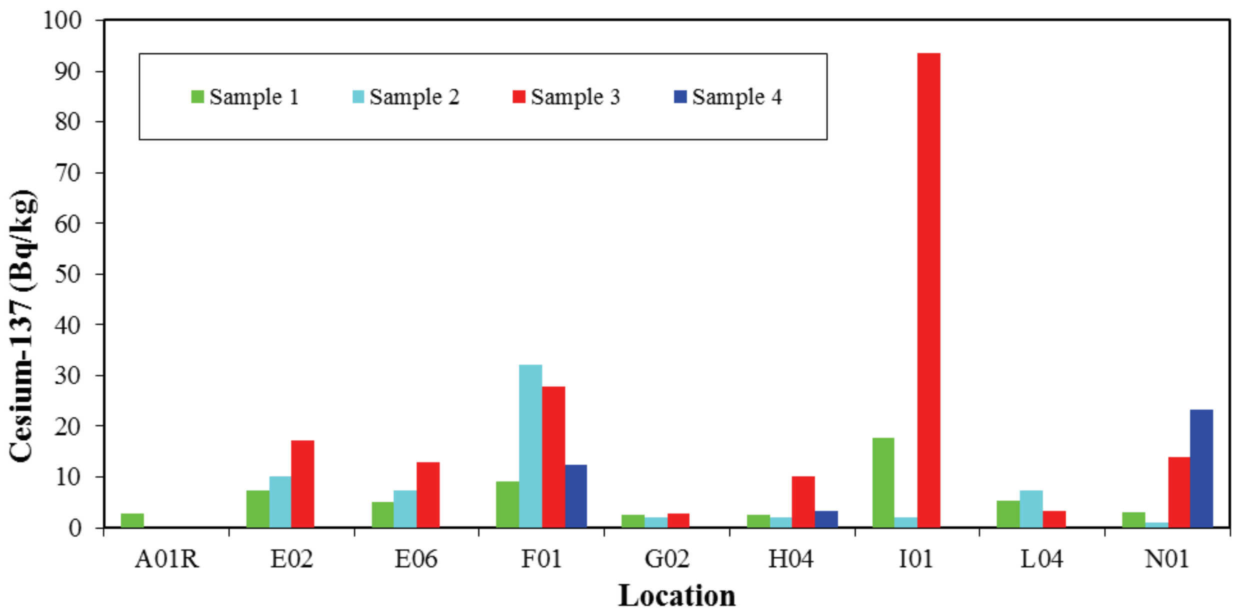


Figure 4.08: Cesium-137 (Soil)

Table 4.12: Soil ($Bq \cdot kg^{-1}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
Cs-137	28	1.2E+0	6.0E-1 to 1.8E+0	1.7E+1	19/27	9.4E-1 to 9.4E+1	*	*	*
Ac-228	28	4.2E+0	2.1E+0 to 1.3E+1	3.2E+1	18/27	1.6E+1 to 5.5E+1	7.0E+1	1/1	7.0E+1 to 7.0E+1
K-40	28	1.8E+1	5.6E+0 to 1.1E+2	5.9E+2	26/27	3.4E+2 to 8.2E+2	1.2E+3	1/1	1.2+3 to 1.2E+3

*The activity is less than or equal to the Critical Level (99% Confidence Level).

4.14 Precipitation

Of the 29 samples analysed, H-3 was detected in 20. The H-3 results are attributable to the operation of PLNGS.

Precipitation is collected continuously at the four onsite air monitoring stations (locations shown in Figure 3.02). The samples are changed approximately monthly, depending on rainfall and freeze up.

Detected H-3 levels spanned 1.7E+01 to 4.3E+02 $Bq \cdot L^{-1}$. Samples taken during periods of heavy rainfall have lower H-3 levels due to dilution.

Since C-14 emissions are low, the expected concentration of C-14 in water is below the detection limit. If the C-14 weekly emissions are more than one percent of the weekly DRL, a C-14 analysis is performed on water. Since the C-14 emissions in 2015 were 1E-03% DRL (and in no week exceeded one percent of the weekly DRL), no further analyses were required. Figure 4.06 shows the weekly C-14 emissions.

Table 4.13 is a summary of the detected radionuclides in precipitation. Figures 4.03 and 4.04 show average monthly H-3 results and Figure 4.05 shows gaseous H-3 emission. “Less Than” values are plotted for non-detected results.

Table 4.13: Precipitation ($Bq \cdot L^{-1}$)

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
H-3	29	1.3E+1	1.2E+1 to 1.4E+1	1.0E+2	20/29	1.7E+1 to 4.3E+2	*	*	*

*There is no reference location.

4.15 Monitoring Well Water, Near Plant

Low levels of H-3 were detected in all samples analysed. These results are attributable to PLNGS emissions.

Eleven monitoring wells are sampled once per year. This frequency will be increased for some or all wells if H-3 concentrations greater than 7000 Bq·L^{-1} are detected. As well, additional samples may be collected if an abnormal release is suspected or an elevated result is obtained.

Tritium concentrations averaged 1.1E+02 Bq·L^{-1}, ranging up to 2.4E+02 Bq·L^{-1}.

Table 4.14 is a summary of the detected radionuclides in monitoring well water. Figure 4.09 shows individual H-3 results. "Less Than" values are plotted for non-detected results.

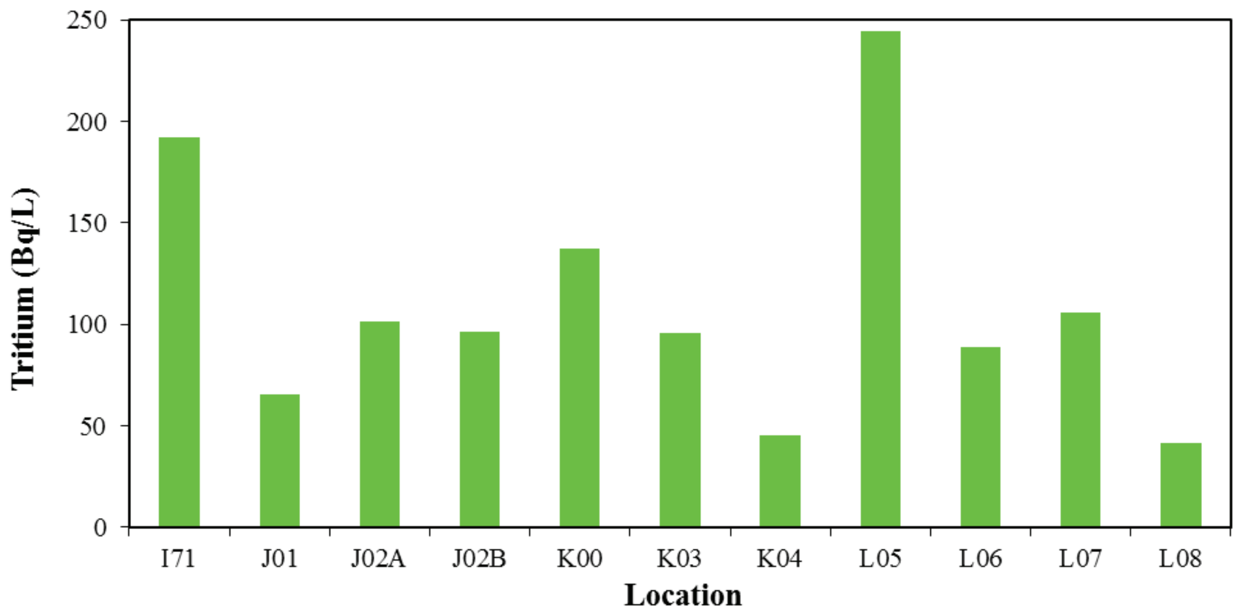


Figure 4.09: Tritium (Monitoring Well Water, Near Plant)

Table 4.14: Monitoring Well Water, Near Plant (Bq·L^{-1})

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
H-3	11	1.3E+1	1.2E+1 to 1.4E+1	1.1E+2	11/11	4.2E+1 to 2.4E+2	*	*	*

*There is no reference location.

4.16 Seawater

Potassium-40 was detected in 19 of the 21 samples analysed for gamma emitters. Tritium was detected in two of the 17 samples analysed from H-3. The H-3 results are attributable to the operation of PLNGS.

Seawater is collected quarterly from three locations close to PLNGS and one reference location near Saint John (shown in Figure 3.05.

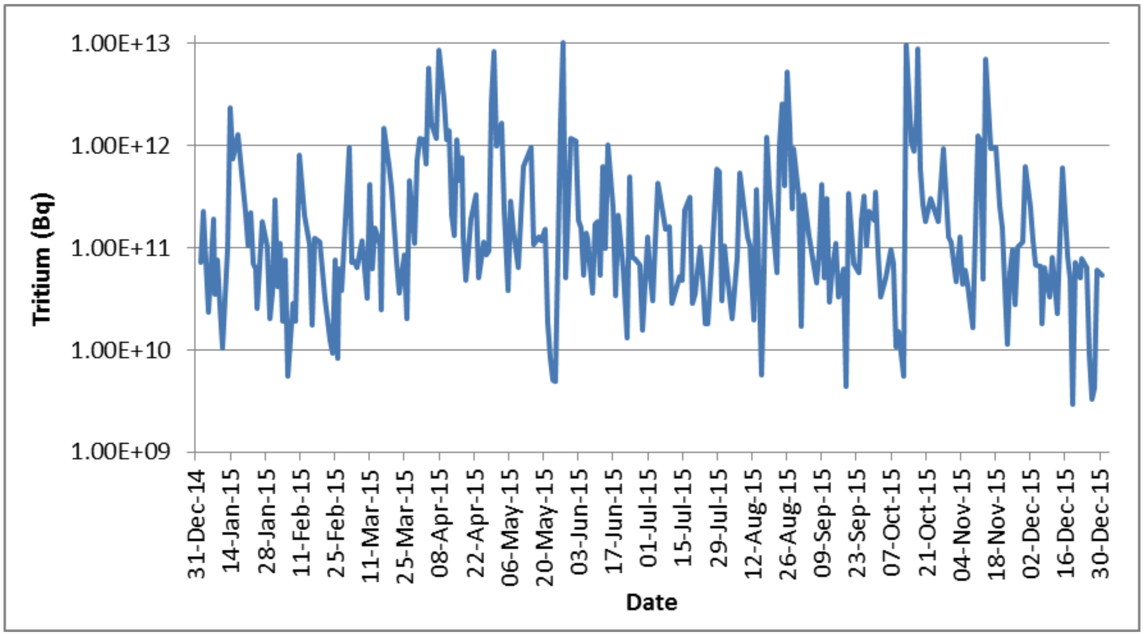
Naturally occurring K-40 was detected (9.1E+00 to 1.4E+01 $Bq \cdot L^{-1}$) in 19 samples. Tritium was detected in two (4.8E+01 to 5.9E+01 $Bq \cdot L^{-1}$). Calculations suggest that the 2015 average concentration of tritium in seawater, due to emissions from PLNGS in the liquid pathway (see Figure 4.10), would be about 1E+01 $Bq \cdot L^{-1}$ at the out-fall (samples are not collected at this point, but are taken at the shoreline nearby). This calculation takes into account the total tritium released over the year, the flow rate of the condenser cooling water (about 2.5E+01 $m^3 \cdot s^{-1}$), and tidal mixing. A dilution factor of 20 is assumed for tidal mixing at the out-fall during normal coolant flows. For collection further away from the outfall, a tidal mixing factor of 40, or even higher, is more realistic. A factor of 40 would result in an average H-3 concentration of about 7E-01 $Bq \cdot L^{-1}$ in seawater during 2015 at the H03 location. In past years, when samples were taken soon after pump out of higher than usual amounts of H-3, the results were much less than the predicted levels. These results further confirm the conservatism in the calculation.

When C-14 and Sr-89,90 emissions are low, the expected concentration of these radionuclides in seawater is below the detection limit. If the monthly emissions are more than one percent of the monthly DRL, a C-14 or Sr-89,90 analysis is performed on seawater. Since the emissions in 2015 were 3E-03% DRL for C-14 and 2E-09% DRL for Sr-90 (and in no month exceeded one percent of the monthly DRL), no further analyses were required. Strontium-89 was not detected in releases. Figure 4.11 shows the monthly C-14 emissions.

Table 4.15 is a summary of the detected radionuclides in seawater.

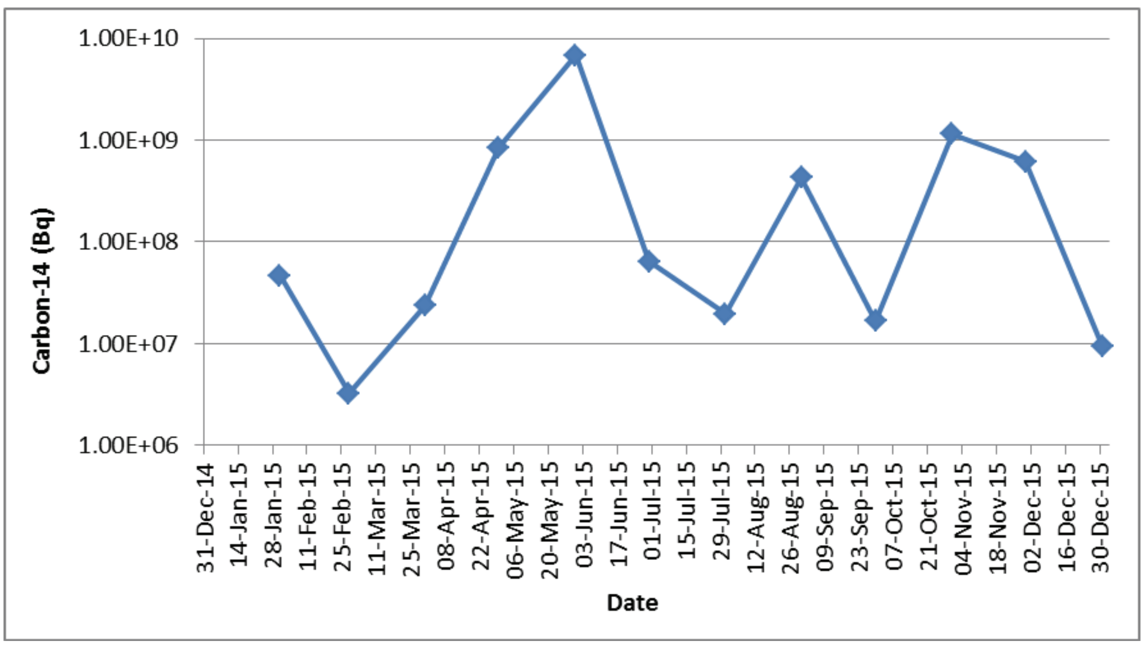
4.17 Tritium and C-14 Analyses of Seafood

When H-3 and C-14 emissions are low, the expected concentrations of these radionuclides in seafood are below the detection limit. If the monthly emissions are more than one percent of the monthly DRL, a H-3 or C-14 analysis is performed on seafood. Since the emissions in 2015 were 3E-04% DRL for H-3 and 3E-03% DRL for C-14 (and in no month exceeded one percent of the monthly DRL), no further analyses were required. Figures 4.10 and 4.11 show the emissions of these radionuclides.



Note: The Monthly DRL for H-3 is 3.8E+18 Bq

Figure 4.10: Liquid H-3 Emissions for 2015



Note: The Monthly DRL for C-14 is 2.7E+13 Bq

Figure 4.11: Liquid C-14 Emissions for 2015

Table 4.15: Seawater ($Bq \cdot L^{-1}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	17	1.3E+1	1.2E+1 to 1.4E+1	5.4E+1	2/13	4.8E+1 to 5.9E+1	*	*	*
K-40	21	2.4E+0	1.3E+0 to 6.7E+0	1.2E+01	14/16	9.3E+0 to 1.4E+1	1.1E+01	5/5	9.1E+0 to 1.3E+1

* The activity is less than or equal to the Critical Level (99 % Confidence Level).

4.18 Seafood

Potassium-40 is usually detected in these samples. The results are not attributable to the operation of PLNGS. Figure 3.05 shows the locations for most of these samples.

Clams – Five samples were collected from the Lepreau area. The inshore fishery often faces restrictions placed upon the harvesting of shellfish, either for conservation of stocks or because of bacterial contamination or algal blooms. The restrictions affect the availability of these sample types for analysis. Data are shown in Table 4.16.

Crab - As in most years, there was no active crab fishery in the local area. No samples were collected in 2015.

Dulse - Dulse is an edible seaweed that is a popular snack food in the area. Seven samples were collected (five from the Lepreau area and two from Grand Manan). Data are shown in Table 4.17.

Fish - The fish category now tends to be made up of haddock and halibut, if they are available at all. Seven samples were collected in 2015 (one from the Lepreau area and six from unknown locations in the Bay of Fundy). Data are shown in Table 4.18.

Lobster – Six samples were collected from the Lepreau area. A few lobster are obtained during each of the two federally regulated fishing seasons per year. Data are shown in Table 4.19.

Periwinkles – Ten samples were collected from the Lepreau area and one from Bocabec. Data are shown in Table 4.20.

Aquaculture Salmon - The aquaculture salmon industry is important to the area west of PLNGS. Late in 2004, a new facility close to the PLNGS outfall in Duck Cove began operation. The fish were removed from here in 2007, and were restocked in 2008. No samples were available in 2015 from this facility, so two samples were collected from Beaver Harbour. Data are shown in Table 4.21.

Scallops - The inshore fishery often faces restrictions or total bans placed upon the harvesting of scallops for the conservation of stocks. One sample was collected from Dipper Harbour and one from an unknown location in the Bay of Fundy. Data are shown in Table 4.22.

4.19 Other Sea Plants

Potassium-40 was detected in all 13 samples analysed. These results are not attributable to the operation of PLNGS.

Sea plants other than dulse are analysed. Various species of seaweed (for example, *Ascophylum*) occur on the rocks on the Point Lepreau peninsula and are collected quarterly. Sample locations are shown in Figure 3.05.

Naturally occurring K-40 ranged from 1.6E+02 to 3.3E+02 $Bq{\cdot}kg^{-1}$. Data are shown in Table 4.23.

Table 4.16: Clams, Edible, Raw Mass (Bq·kg^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	5	1.3E+2	4.1E+1 to 2.8E+2	2.1E+2	3/5	2.1E+2 to 2.2E+2	*	*	*

*There were no reference samples.

Table 4.17: Dulse, Wet Mass (Bq·kg^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	7	9.3E+1	1.1E+1 to 2.1E+2	6.3E+2	5/5	3.3E+2 to 9.5E+2	5.3E+2	2/2	5.2E+2 to 5.4E+2

Table 4.18: Fish, Raw Mass (Bq·kg-1)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	7	2.8E+1	4.6E+0 to 8.1E+1	1.2E+2	6/7	7.5E+1 to 1.6E+2	*	*	*

*There were no reference samples.

Table 4.19: Lobster, Edible, Cooked Mass (Bq·kg^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	6	1.9E+1	1.4E+1 to 2.3E+1	1.0E+2	6/6	7.0E+1 to 1.5E+2	*	*	*

* There are no reference samples.

Table 4.20: Periwinkles, Edible, Raw Mass (Bq·kg^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	11	7.1E+1	2.8E+1 to 1.2E+2	1.3E+2	5/10	9.7E+1 to 1.7E+2	*	*	*

*There were no reference samples.

Table 4.21: Aquaculture Salmon, Raw Mass (Bq·kg^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
K-40	2	5.4E+1	1.0E+1 to 9.7E+1	1.7E+2	2/2	1.4E+2 to 2.0E+2	*	*	*

*There were no reference samples.

Table 4.22: Scallops, Raw Mass ($Bq \cdot kg^{-1}$)

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
K-40	2	2.5E+1	8.2E+0 to 4.3E+2	8.6E+1	2/2	7.9E+1 to 9.2E+1	*	*	*

*There were no reference samples.

Table 4.23: Sea Plants, Wet Mass (Bq·kg^{-1})

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
K-40	13	2.3E+1	1.8E+0 to 6.6E+1	2.4E+2	13/13	1.6E+2 to 3.3E+2	*	*	*

*There were no reference samples.

4.20 Sediment

Of the 46 samples analysed, Be-7 was detected in six, Ac-228 in 29 and K-40 in 46 samples. None of these results are attributable to the operation of PLNGS.

Sediments are collected quarterly from ten locations shown in Figure 3.05. The finer grains are analysed by selective sieving of the material.

All samples contained K-40 (4.7E+02 to 1.4E+03 $Bq \cdot kg^{-1}$) from the natural potassium in feldspar, a common mineral. Six samples contained cosmogenically produced Be-7 (1.6E+01 to 3.1E+01 $Bq \cdot kg^{-1}$). Twenty-eight samples contained Ac-228 (1.0E+01 to 5.2E+01 $Bq \cdot kg^{-1}$), a radioactive progeny of naturally occurring Th-232. Sediment samples analysed between 1977 and 1982, before PLNGS began operations, contained an average Cs-137 concentration of 5.0E+00 $Bq \cdot kg^{-1}$. A small additional Cs-137 component was added to this reservoir from Chernobyl in 1986 and from Fukushima in 2011. Finer grain sediments have a higher natural radioactivity content than coarse sediments.

Table 4.24 is a summary of the detected radionuclides in sediment.

Table 4.24: Sediment (Bq·kg^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
Be-7	46	1.0E+1	5.3E+0 to 2.4E+1	2.3E+1	4/42	1.7E+1 to 3.1E+1	2.0E+1	2/4	1.6E+1 to 2.3E+1
Ac-228	46	3.8E+0	1.7E+0 to 1.4E+1	2.3E+1	25/42	1.0E+1 to 5.2E+1	2.0E+1	4/4	1.2E+1 to 2.4E+1
K-40	46	2.7E+1	4.5E+0 to 1.6E+2	6.7E+2	42/42	4.7E+2 to 1.4E+3	5.9E+2	4/4	5.1E+2 to 6.5E+2

4.21 Ambient Gamma Measurements of Intertidal Zone (Ion Chamber)

Environmental gamma survey measurements are made in the intertidal zone on beaches in the Lepreau area and at the reference location 28 km to the east-northeast (Figure 3.05). Although TLDs are preferred in measuring such exposures since they span the entire year and provide integrated measurements, they cannot be used in these locations. Instead, beach surveys are performed and grab samples of sediments are analysed. Radiation values measured in 2015 were consistent with those measured prior to station start-up in 1982. These values are summarised in Table 4.25.

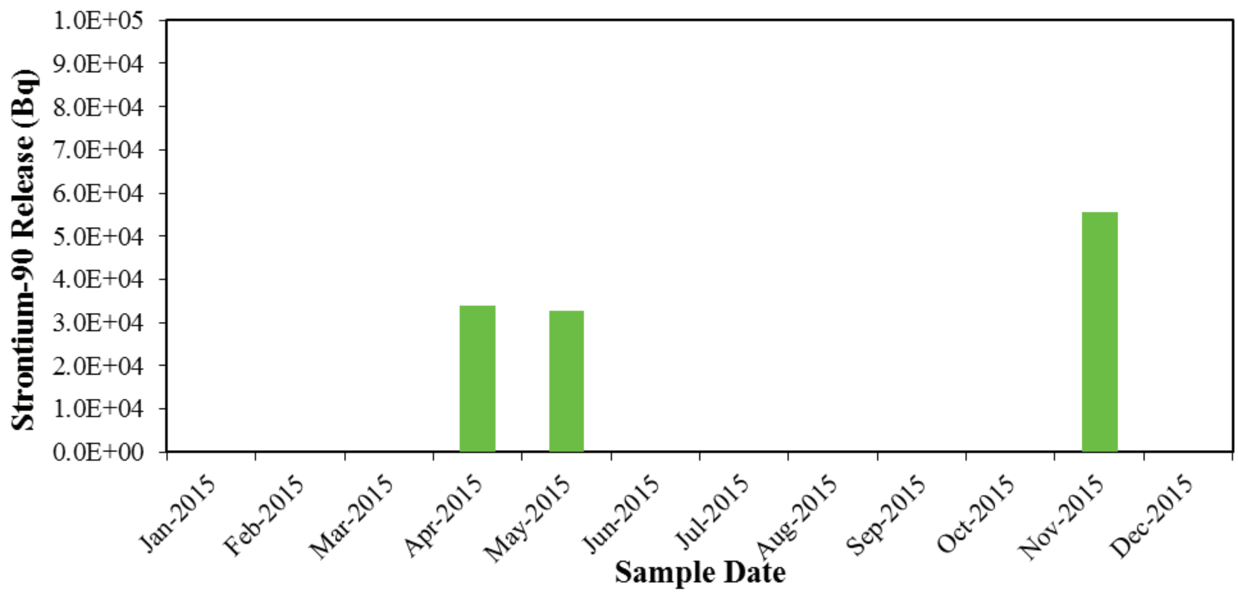
Table 4.25: Ambient Gamma Measurements of Intertidal Zone (Ion Chamber) – (μSv·h^{-1})

Location	*1st Quarter*	*2nd Quarter*	*3rd Quarter*	*4th Quarter*
B01	0.17	0.15	0.16	0.13
B02	N/A	0.19	0.17	0.22
C01	N/A	0.15	0.18	0.11
D04	0.20	0.19	0.14	0.25
E03	0.16	0.15	0.13	0.07
G01	0.15	0.19	0.13	0.16
H03	N/A	0.16	0.13	0.17
N04	0.19	0.20	0.14	0.14
P02	N/A	0.17	0.11	0.22
Q01R	0.20	0.18	0.16	0.15

NA: Data Not Available

4.22 LEM Composite Water (Sr-89,90)

When Sr-89,90 emissions are low, the expected concentration of Sr-89,90 in seawater is below the detection limit. The LEM collects samples of PLNGS liquid emissions at their source. A monthly composite is sent to the lab for analysis. Twelve of these composites were analysed for Sr-89,90. If the monthly emissions are more than one percent of the monthly DRL, a Sr-89,90 analysis is performed on seawater. Since the emissions in 2015 were 2E-09% DRL (and in no month exceeded one percent of the monthly DRL) for Sr-90, and Sr-89 was not detected, no further analyses were required. Figure 4.12 shows the Sr-90 emissions from PLNGS.



Note: The Monthly DRL for Sr-90 is 5.0E+14 Bq

Figure 4.12: Liquid Sr-90 Emissions

4.23 Bore Hole Water, SRWMF

Of the 100 samples analysed, low levels of H-3 were detected in 92. The H-3 results are attributable to the operation of PLNGS.

Samples are taken three times per year from 35 drilled wells. Occasionally, a well is dry or inaccessible and no sample is available.

Tritium concentrations averaged 1.4E+02 Bq·L^{-1} (3.4E+01 to 4.0E+02 Bq·L^{-1}) near the Phase 1 facility, 5.1E+01 Bq·L^{-1} (1.3E+01 to 5.1E+02 Bq·L^{-1}) near the Phase 2 facility and 1.1E+02 Bq·L^{-1} (2.2E+01 to 4.2E+02 Bq·L^{-1}) near the Phase 3 facility. Tritium washes out into precipitation and subsequently drains into some of the bore holes. Precipitation analyses (Section 4.14) indicate H-3 concentrations ranging from 1.7E+01 to 4.3E+02 Bq·L^{-1} in 20 of 29 samples.

Results are presented in Tables 4.26 to 4.28. Figure 4.13 shows the H-3 activity at each bore hole for each sample. Locations I10A-I10F are the closest to the onsite SRWMF- Phase 1 structures and may be affected by the elevated H-3 levels associated with the structures. “Less Than” values are plotted for non-detected results.

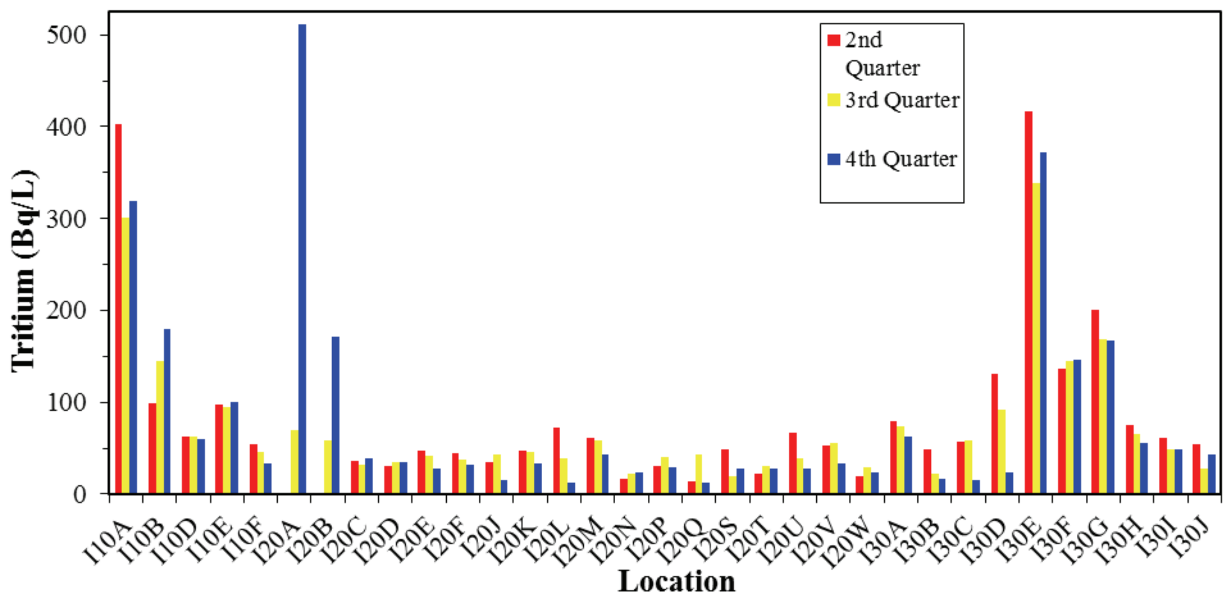


Figure 4.13: Tritium (Bore Hole Water, SRWMF)

Table 4.26: Bore Hole Water, SRWMF - Phase 1 ($Bq \cdot L^{-1}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	15	1.2E+1	1.2E+1 to 1.3E+1	1.4E+2	15/15	3.4E+1 to 4.0E+2	*	*	*

*There is no reference location.

Table 4.27: Bore Hole Water, SRWMF - Phase 2 (Bq·L^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	52	1.2E+1	1.2E+1 to 1.3E+1	5.1E+1	47/52	1.3E+1 to 5.1E+2	*	*	*

*There is no reference location.

Table 4.28: Bore Hole Water, SRWMF - Phase 3 (Bq·L^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	33	1.3E+1	1.2E+1 to 1.4E+1	1.1E+2	30/33	2.2E+1 to 4.2E+2	*	*	*

*There is no reference location.

4.24 Parshall Flume Water, SRWMF

Of the 160 samples analysed, H-3 was detected in 138. These results are attributable to the emissions from PLNGS and the material stored in the Phase 1 structures.

Rainwater and snow melt at the SRWMF (Phases 1, 2 and 3) are obtained from drainage channels (flumes) constructed to collect surface runoff from these areas. Samples are collected and analysed on a weekly basis.

There is little or no flow into or out of these collection locations during the winter months and values for H-3 tend to vary little from one week to the next except after heavy rain. The average H-3 value for each phase is:

- 2.5E+02 Bq·L^{-1} at Phase 1
- 6.8E+01 Bq·L^{-1} at Phase 2
- 6.9E+01 Bq·L^{-1} at Phase 3.

The Phase 1 results are due to H-3 vapour escaping from the structures and condensing onto surfaces. Tables 4.29 to 4.31 are summaries of the detected radionuclides in the flumes. Figure 4.14 compares the H-3 in the samples from the three facilities. "Less Than" values are plotted for non-detected results.

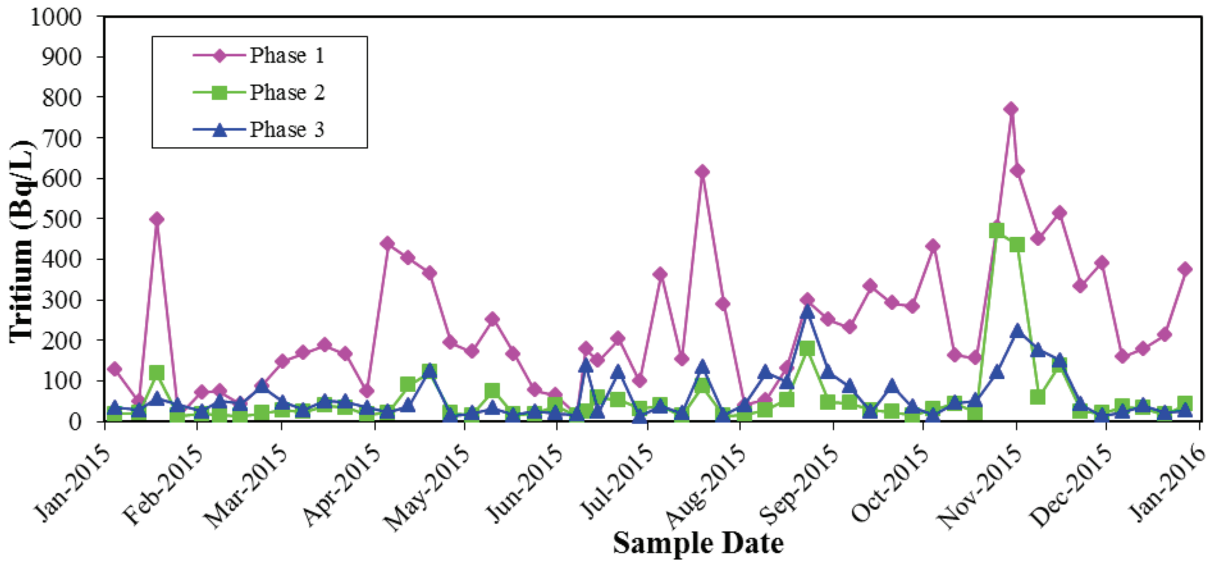


Figure 4.14: Tritium (Parshall Flume Water, SRWMF)

Table 4.29: Parshall Flume Water, SRWMF - Phase 1 ($Bq \cdot L^{-1}$)

Analysis Type	***Total Number***	***Critical Level***		***Indicator Locations***			***Reference Locations***		
		Mean	***Range***	***Mean***	***Frequency***	***Range***	***Mean***	***Frequency***	***Range***
H-3	54	1.3E+1	1.2E+1 to 1.4E+1	2.5E+2	53/54	1.6E+1 to 7.7E+2	*	*	*

*There is no reference location.

Table 4.30: Parshall Flume Water, SRWMF - Phase 2 (Bq·L^{-1})

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	53	1.3E+1	1.2E+1 to 1.4E+1	6.8E+1	40/53	1.4E+1 to 4.7E+2	*	*	*

*There is no reference location.

Table 4.31: Parshall Flume Water, SRWMF - Phase 3 ($Bq \cdot L^{-1}$)

Analysis Type	*Total Number*	*Critical Level*		*Indicator Locations*			*Reference Locations*		
		Mean	*Range*	*Mean*	*Frequency*	*Range*	*Mean*	*Frequency*	*Range*
H-3	53	1.3E+1	1.1E+1 to 1.4E+1	6.9E+1	45/53	1.4E+1 to 2.7E+2	*	*	*

*There is no reference location.

4.25 Hemlock Knoll Regional Sanitary Landfill Program

PLNGS disposes of its non-active waste at the public landfill facility at Hemlock Knoll. The monitoring program consists of dosimeter placement at key locations.

There were 16 TLD results from Hemlock Knoll in 2015. TLD results appear in Table 4.06 (location codes YTL1 to YTL4).

4.26 Meteorological Data

The meteorological data for 2015 were collected at ten minutes intervals and are presented in Table 4.32. Wind Rose data for 2015 are presented in Figure 4.15.

Table 4.32: Meteorological Data for Point Lepreau (2015)

	Temperature (Degrees Celsius) 10 Metre Tower Data					Wind Direction* (Relative %) 42 Metre Tower Data							
		Mean Daily		Extreme		% Observations from							
Month	**Avg**	**Max**	**Min**	**Max**	**Min**	**N**	**NE**	**E**	**SE**	**S**	**SW**	**W**	**NW**
January	-6.3	-1.6	-12.1	8.8	-23.3	17	9	2	3	5	22	27	15
February	-10.5	-5.3	-15.6	3.2	-22.5	22	4	2	5	6	16	29	16
March	-2.7	1.4	-7.1	10.4	-20.2	8	5	2	10	17	16	31	12
April	2.9	6.7	-0.2	13.3	-10.0	15	14	6	10	15	12	15	13
May	8.5	12.9	5.3	23.0	-0.4	11	11	5	24	22	14	9	4
June	12.0	15.9	8.8	26.9	5.7	16	11	6	21	17	11	7	10
July	15.3	19.8	12.2	25.8	9.6	13	19	11	18	14	13	7	6
August	16.8	20.9	14.1	26.9	11.8	11	13	10	20	24	17	3	2
September	15.6	19.8	12.1	30.0	5.4	12	6	3	16	23	17	10	12
October	8.8	12.5	4.6	17.6	-2.3	15	6	6	13	16	11	22	11
November	5.4	8.5	2.1	14.9	-10.8	13	7	5	10	10	16	23	17
December	2.6	5.7	-0.7	11.1	-13.4	16	9	7	8	8	16	19	17
Average for 2015	5.7	Max 9.8	Min 2.0	Extreme Max 30.0	Extreme Min -23.3	14	10	5	13	15	15	17	11

*Each compass direction covers ±22.5 degrees.

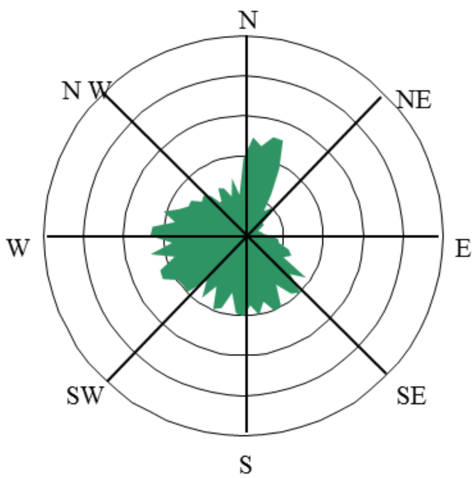


Figure 4.15: Wind Rose for Point Lepreau (2015)

5 Trends (REMP)

The following trends were observed in the historical data:

- Gaseous tritium and C-14 emissions remained low in 2015.
- The concentration of tritium in Parshall Flume water from the Solid Radioactive Waste Management Facility (SRWMF) Phase 1 has been decreasing since 2005.
- The dose to the Representative Person remained low in 2015.
- Tritium continues to be detected in air and water samples (higher onsite than offsite).
- There continues to be a difference between onsite and offsite thermoluminescent dosimeter (TLD) measurements (elevated onsite compared with offsite).
- The radionuclide concentration in most sample types continues to remain at preoperational (background) levels due to the history of low emissions.
- The concentration of tritium in a monitoring well that is close to the reactor building had been increasing since 2005, but has dropped dramatically since 2009.

As in the figures in Section 4, "Less Than" values are plotted for non-detected values. All location codes are described in Appendix C.

5.01 Dose from Airborne and Liquid Pathways

Radiation dose from PLNGS emissions continues to be well below the public dose limit (1000 microsieverts per annum), and also well below the design and operating target for PLNGS (50 microsieverts per annum). The dose to the Representative Person was lower in 2010 and 2011 due to the refurbishment outage (heavy water systems were laid up and reactor components were removed). The increase in 2012 was due to restart activities. See Figure 5.01. Elevated doses from liquid effluents are associated with prolonged outages.

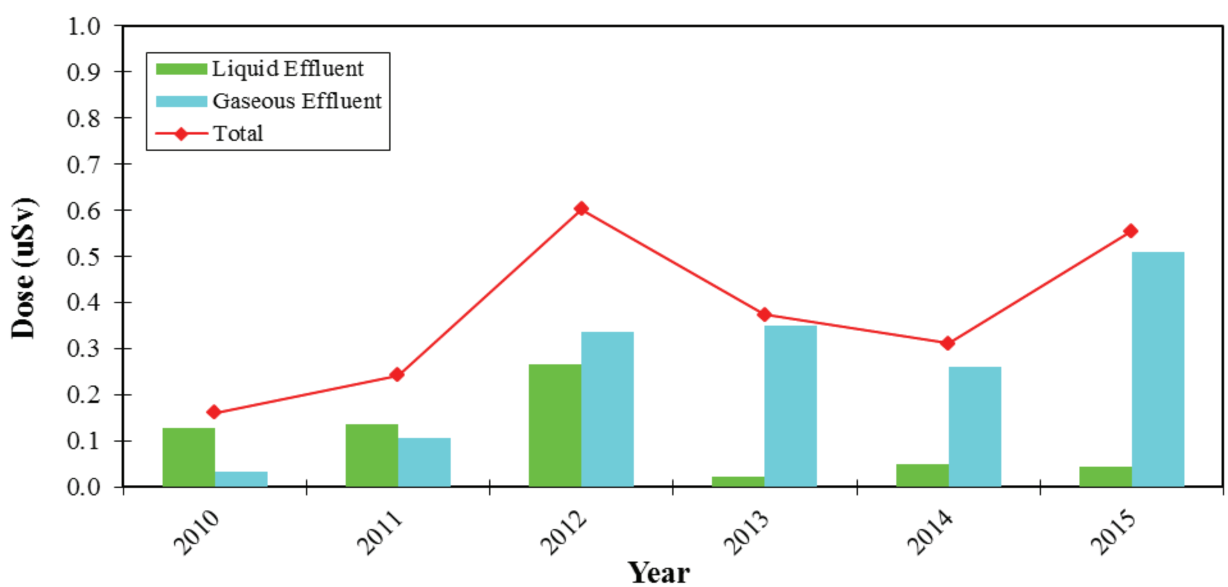
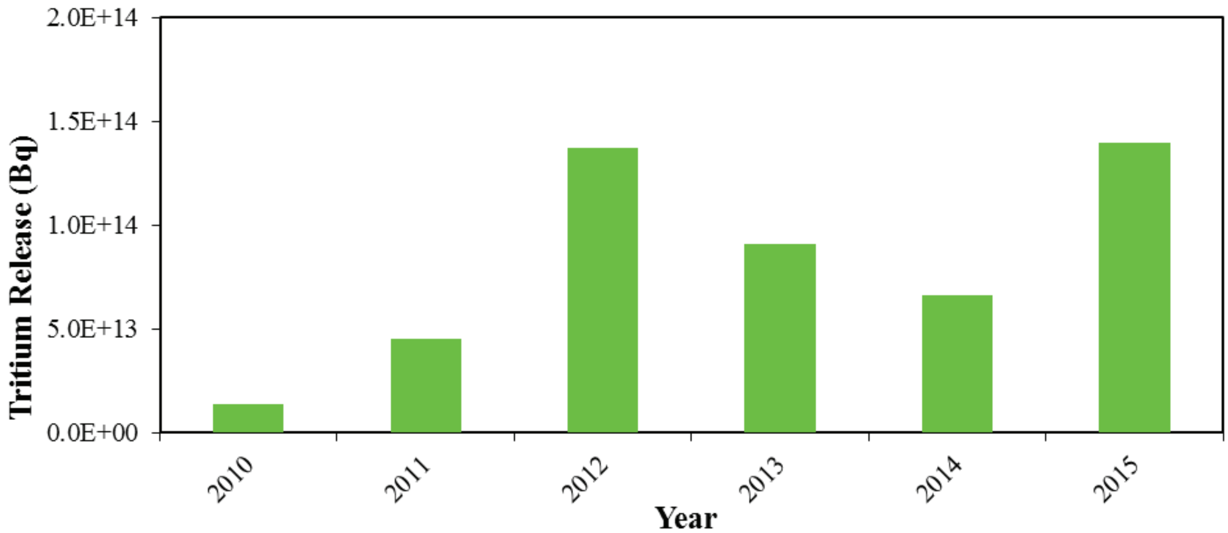


Figure 5.01: Dose from Airborne and Liquid Pathways

5.02 Tritium (Water Vapour)

Due to the refurbishment outage, Station airborne tritium emissions were at a historic low level in 2010 (Figure 5.02). The increase in 2012 was due to restart activities. The onsite stations show a similar decline (Figures 5.04) in airborne H-3 concentration and the offsite locations continue to be low (Figure 5.03). This concentration gradient is due to increasing dilution with distance from the emissions stack.



Note: The current Annual DRL for H-3 is 2.8E+17 Bq

Figure 5.02: Airborne H-3 Emissions

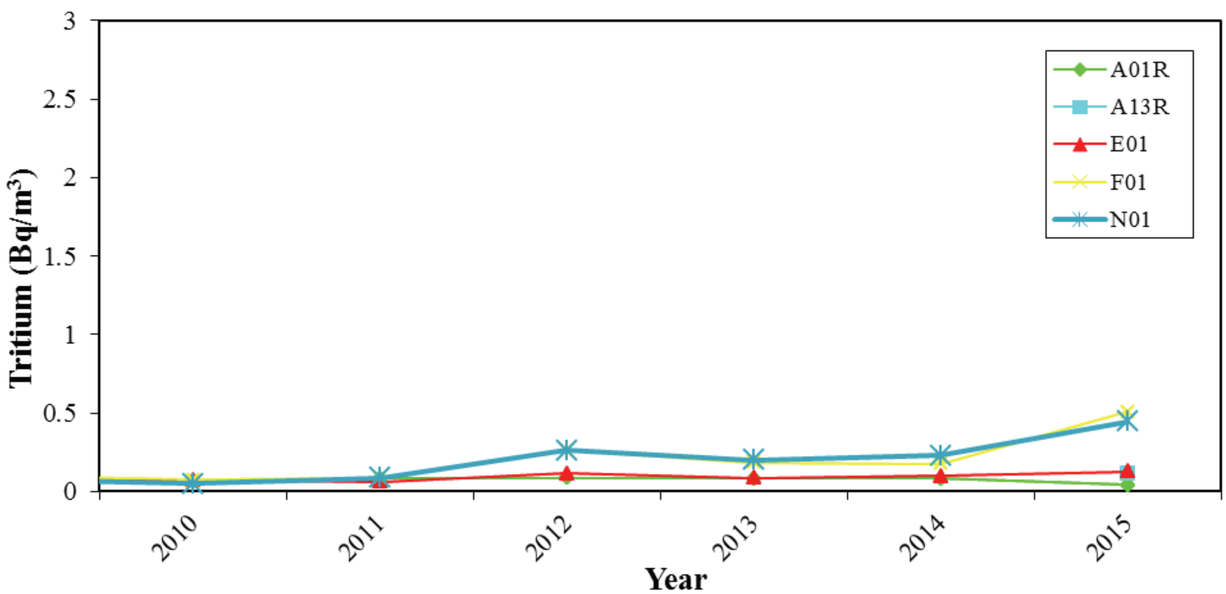


Figure 5.03: Tritium (Water Vapour) at Offsite Air Stations

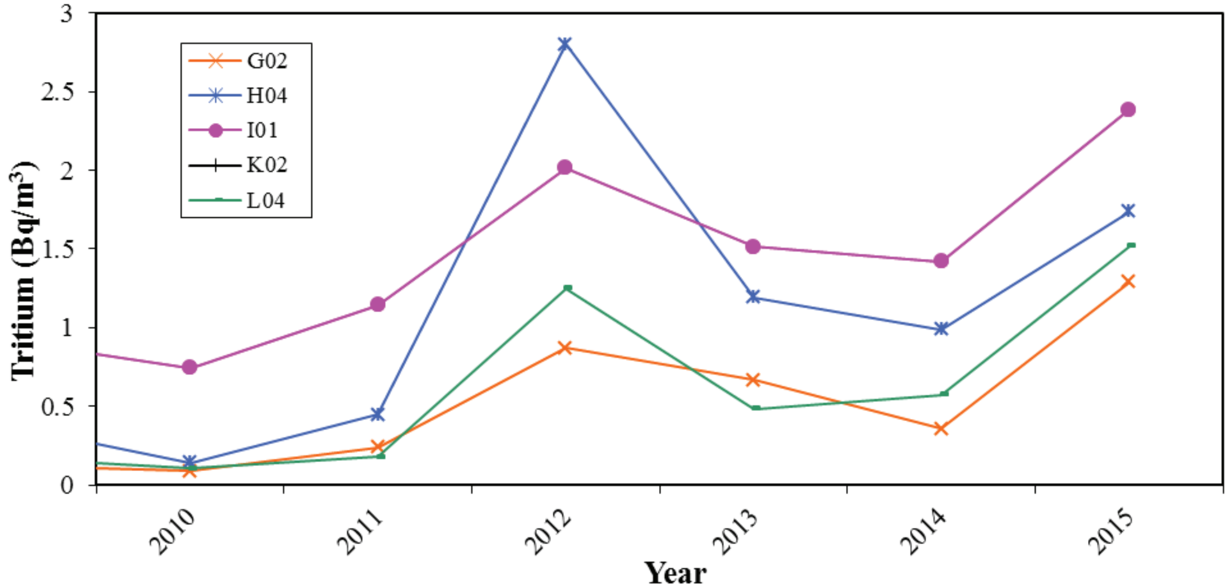


Figure 5.04: Tritium (Water Vapour) at Onsite Air Stations

5.03 Cesium-137 (Soil)

Cesium-137 from the fallout of past atmospheric weapons tests and international events tends to accumulate in the organic layer of soil. There can be large fluctuations in Cs-137 levels due to this organic load in the sample.

The value plotted for each year in Figure 5.05 is the mean of all values for that year. "Less Than" values are plotted for non-detected values.

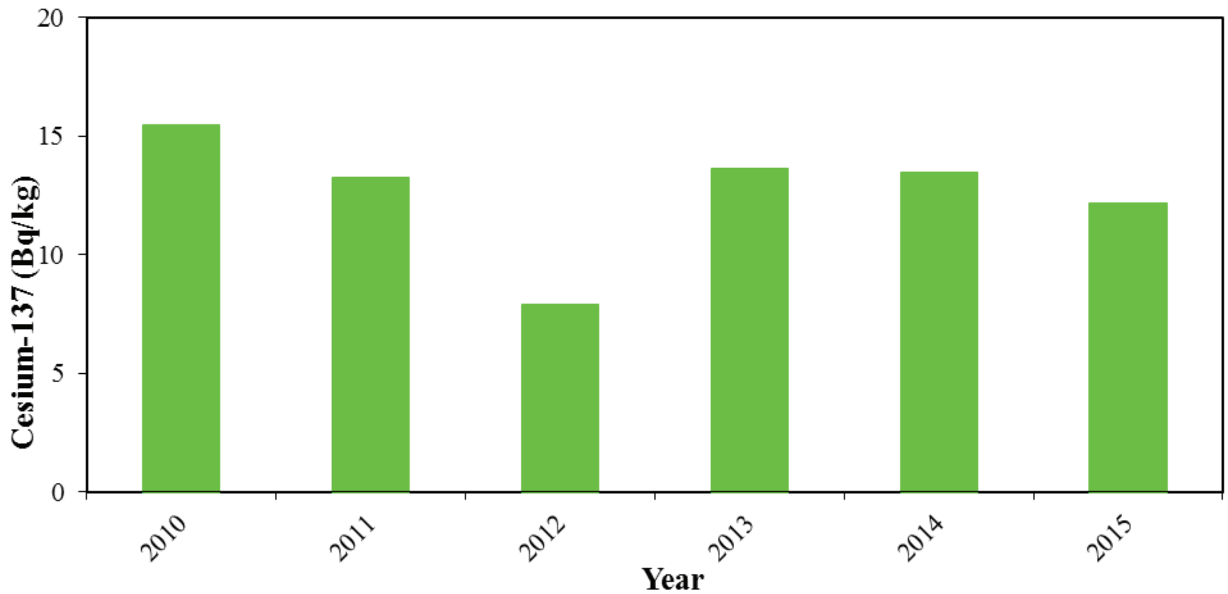


Figure 5.05: Cesium-137 (Soil)

5.04 Tritium (Monitoring Well Water, Near Plant)

The concentration of H-3 in the monitoring well at K00, which is close to the reactor building has been decreasing since 2010.

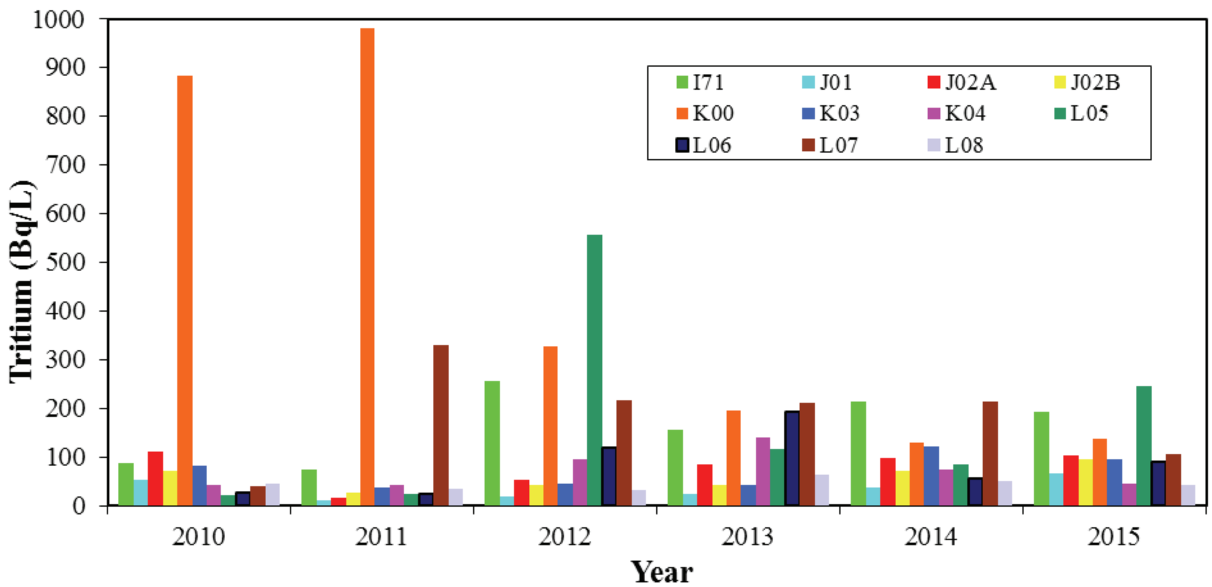


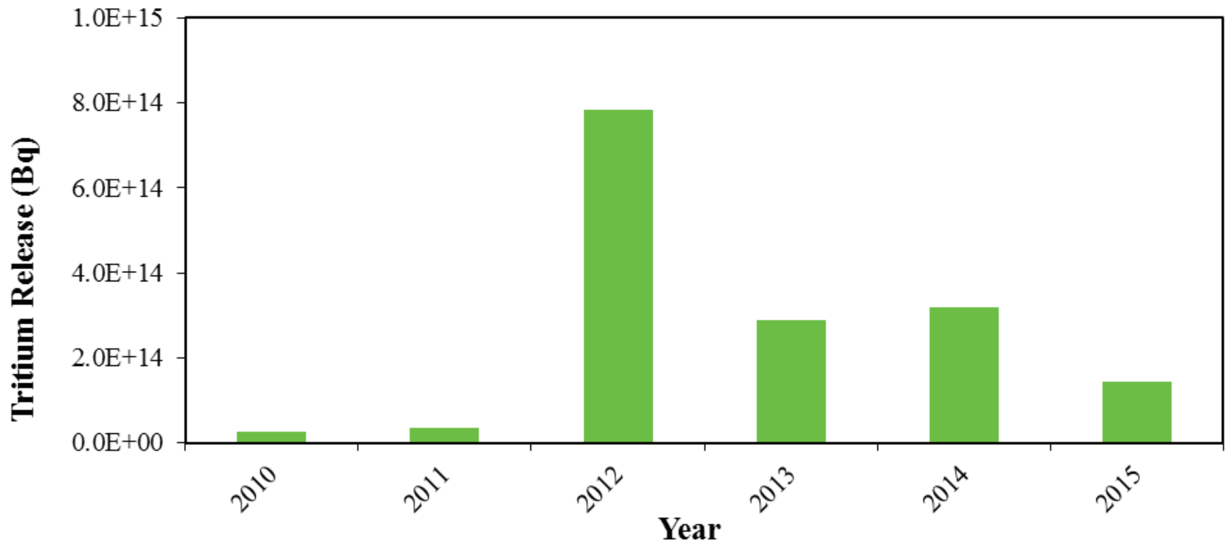
Figure 5.06: Tritium (Monitoring Well Water, Near Plant)

5.05 Tritium and C-14 (Seawater)

Tritium emissions to seawater have been declining since start up activities after the refurbishment outage in 2012 (Figure 5.07). The increase in 2012 was due to restart activities.

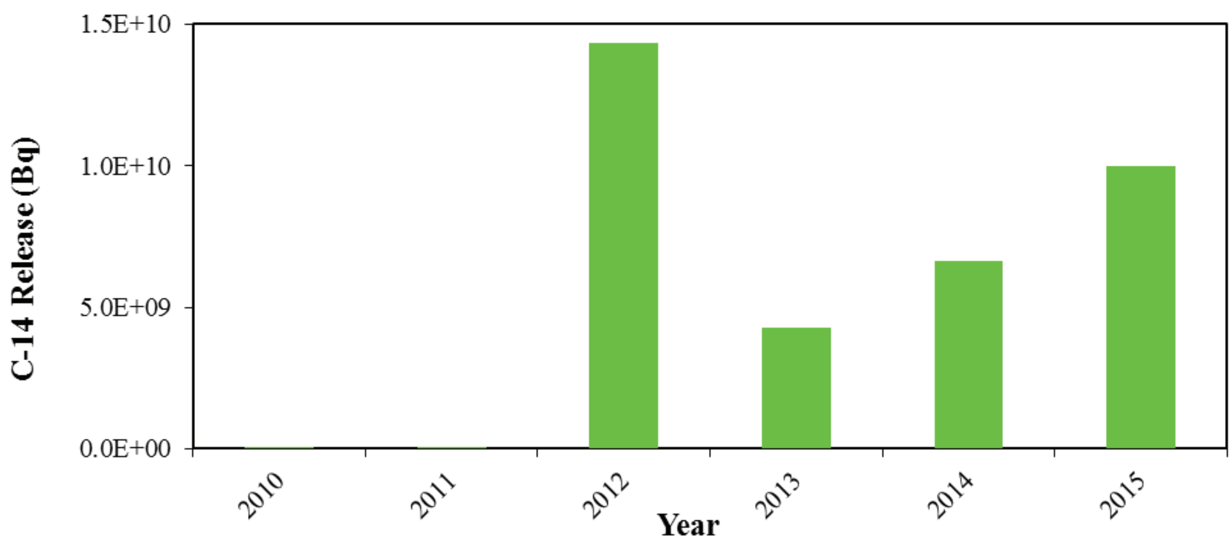
The value plotted for each year in Figure 5.07 is the mean of all values for that year. “Less Than” values are plotted for non-detected values.

Carbon-14 emissions were up in 2012 due to restart activities including the transfer of moderator water to the calandria. The expected concentration of C-14 in seawater is below the detection limit (Figure 5.08).



Note: The current Annual DRL for H-3 is 4.6E+19 Bq

Figure 5.07: Liquid H-3 Emissions

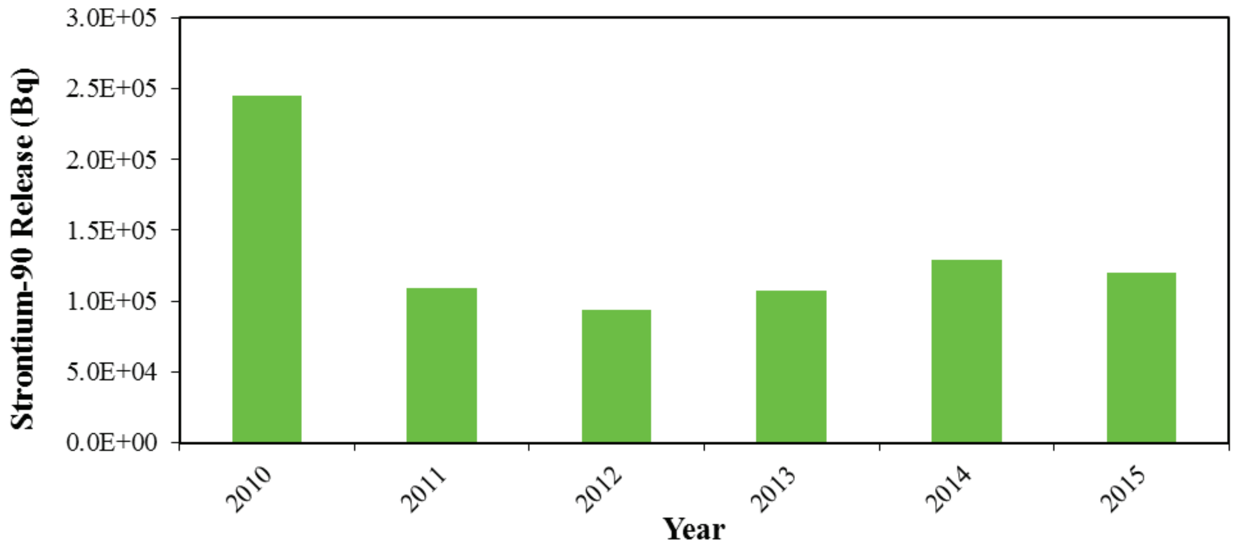


Note: The current Annual DRL for C-14 is 3.3E+14 Bq

Figure 5.08: Liquid C-14 Emissions

5.06 Strontium-90 (LEM Water)

The maximum values for Sr-90 still represent only a small fraction of the DRL and are due to activity just above the detection limit (Figure 5.09).



Note: The current Annual DRL for Sr-90 is 6.0E+15 Bq

Figure 5.09: Liquid Sr-90 Emissions

5.07 Tritium (Parshall Flume Water)

The H-3 values at Phase 2 and Phase 3 are typically less than those at Phase 1. The Phase 1 results are due to H-3 vapour escaping from the structures and condensing onto surfaces (Figure 5.10).

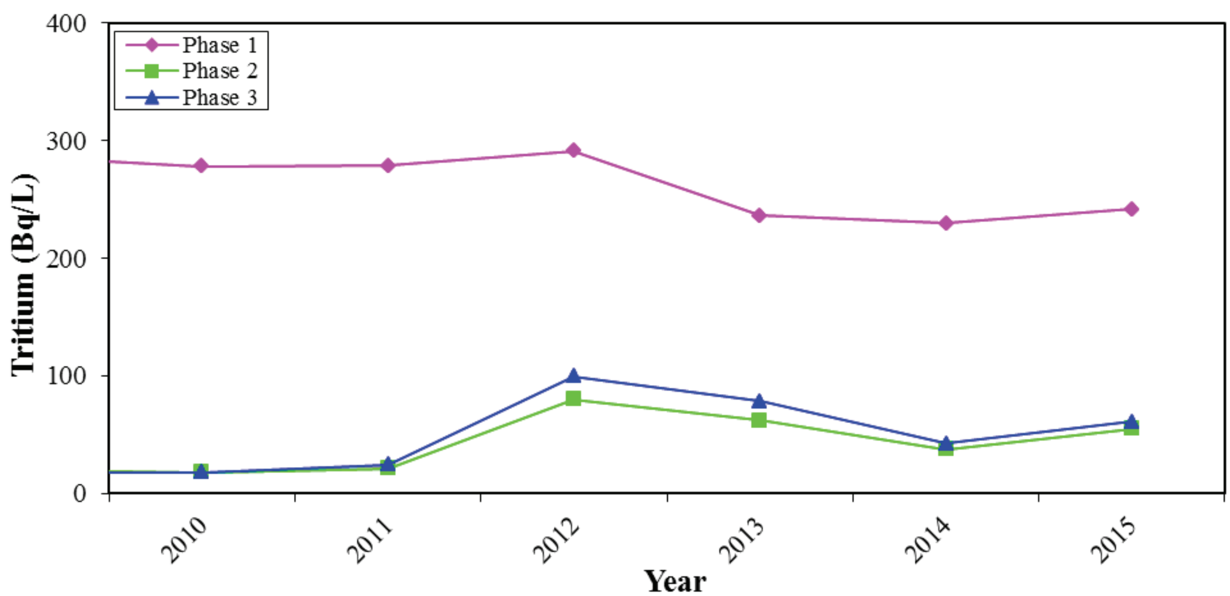


Figure 5.10: Tritium (Parshall Flume Water)

6 Dose Estimation

The DRLs apply to the release point for each of the two major effluent pathways for PLNGS: the ventilation stack for airborne releases; and, for liquid releases, the discharge point of the Condenser Cooling Water (CCW) duct into the Bay of Fundy. The releases are assumed to be continuous. All relevant exposure routes to the public are factored into the DRL calculations. Crossover routes between the two pathways are insignificant, and therefore they are not considered.

The DRL document identifies the Representative Person associated with radioactive airborne and liquid effluent releases from the PLNGS, and documents the magnitude of activity of each nuclide released through either pathway in one calendar year that would cause the Representative Person to receive or be committed to the regulatory dose limit for a member of the public. This activity is called the derived release limit (DRL) for that nuclide.

Dose estimates to members of the local communities that are based on the DRLs are conservative *CSA Standard N288.1-08, Guidelines for calculating derived release limits for radioactive material in airborne and liquid effluents for normal operation of nuclear facilities*, which forms the basis for DRLs, includes conservative values for food intake and other parameters. In some cases, even more conservative site-specific data are used.

The detailed discussion of these pathways may be found in *RD-01364-L1, Derived Release Limits for Radionuclides in Airborne and Liquid Effluents.*

The airborne exposure pathways from PLNGS to the public are:

- internal from inhalation
- external from immersion in a plume
- external from contaminated ground (ground shine)
- internal from ingestion of contaminated well water
- external from immersion in contaminated well water
- internal from ingestion of contaminated soil, plants and animals.

The selection of Representative Person is based upon which local residential areas receive the greatest exposure from airborne releases, and the potential of intakes based upon dietary and behavioral habits.

Welch Cove was selected as the location for the representative group for all airborne releases. Welch Cove is a small community of approximately 32 residences along a two kilometre stretch of road that extends from northwest to north-northwest of PLNGS.

A hypothetical family consisting of two adults, a ten year old child and a one year old infant is considered to be representative of the community.

The liquid exposure pathways from PLNGS to the public are:

- external from diving in contaminated water
- external from exposure to contaminated sediment (while harvesting clams and dulse)
- internal from ingestion of contaminated fish, lobster, clams, and dulse.

- external from diving for sea urchins

The selection of a Representative Person is based upon dietary and behavioral habits of local residents. A representative family of two adults, a ten-year-old child and a one-year-old infant was selected.

The DRLs are based on *CSA Standard N288.1-08, Guidelines for calculating derived release limits for radioactive material in airborne and liquid effluents for normal operation of nuclear facilities*. Station releases of a radionuclide at 100% DRL for a year would result in a dose to the Representative Person of 1000 μSv. In 2015 (Table 6.01), the liquid releases were 4.5E-3% DRL, which corresponds to 0.05 μSv to the Representative Person. Airborne releases for 2015 were 5.1E-2% DRL, which corresponds to a public dose of 0.51 μSv. Adjustments are made to the DRL based on operational considerations or emission location. For example, a reduced CCW flow changes the dilution factor which decreases the DRL.

As shown in Table 6.02 and Figures 6.01 and 6.02, H-3 accounts for 90% of the dose from airborne emissions, and 7% of the dose from liquid emissions in 2015. Iron-59 accounts for 11% of the dose and C-14 accounts for 69% of the dose from liquid emissions

Because of the protective assumptions used in the DRL calculations, and the relatively low level of emissions, the most exposed member of the general public received less than the calculated dose of 0.51 μSv. This radiation dose may be compared with the individual natural radiation dose in the Lepreau area of approximately 2000 to 3000 μSv per annum. (TLDs show only the external, penetrating component, amounting to about 500 to 1000 μSv.) This includes natural dose contributions from ground, air, food and from an assumed low concentration of radon in homes. A significant fraction of Canadian homes contain radon levels that give a much larger radiation dose than the 2000 to 3000 μSv.

Table 6.01: Annual Dose (2015)

Source of Dose to the Representative Person	***Dose to the Representative Person ($\mu Sv.a^{-1}$)***
PLNGS airborne emissions	0.51
PLNGS liquid emissions	0.05

Table 6.02: Contribution of Radionuclides to Dose in Each Pathway (2015)

Radiouclide	***Contribution to Dose (from Airborne Emissions)***	***Contribution to Dose (from Liquid Emissions)***
H-3	94.4 %	6.8 %
C-14	2.0 %	68.9 %
Ar-41	2.9 %	----
Fe-59	----	11.3 %
Co-60	----	8.9 %
Zr-95	----	2.2 %
Sn-113	----	1.7 %
All others	0.7 %	0.2 %
TOTAL	***100 %***	***100 %***

Note: Only radionuclides contributing 0.5% or more are itemized.

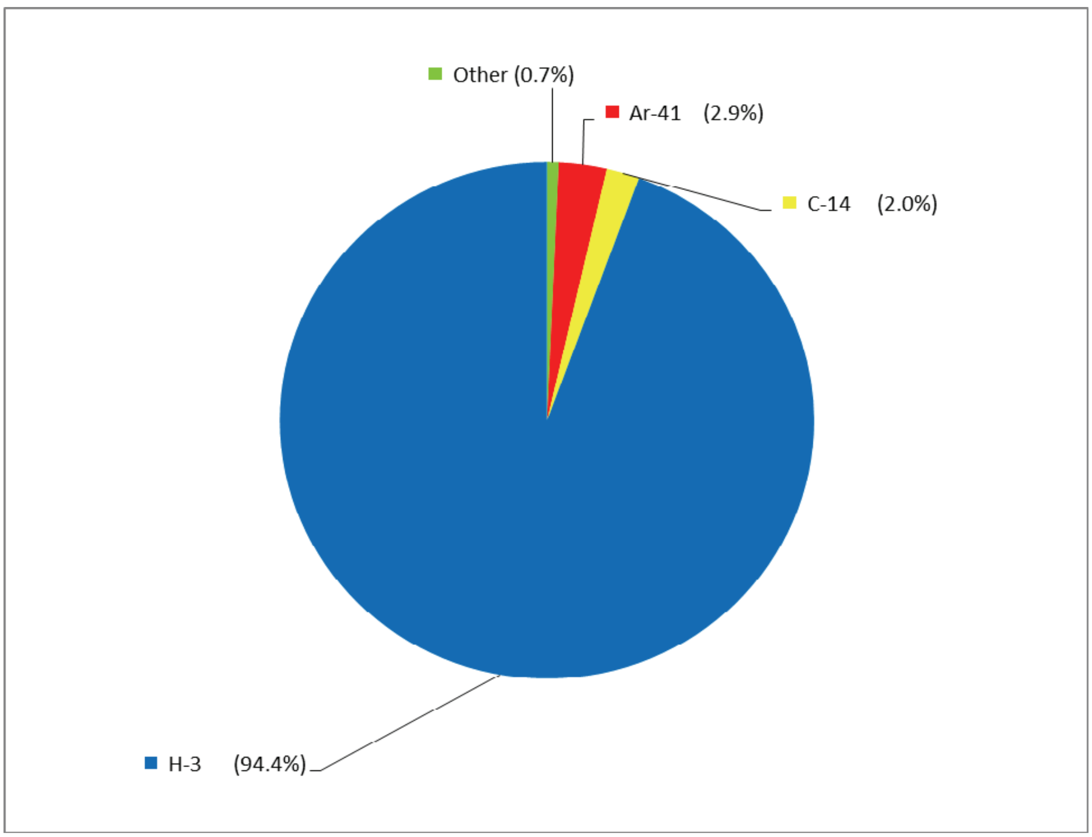


Figure 6.01: Contribution of Radionuclide to Total Dose (Airborne Pathway)-2015

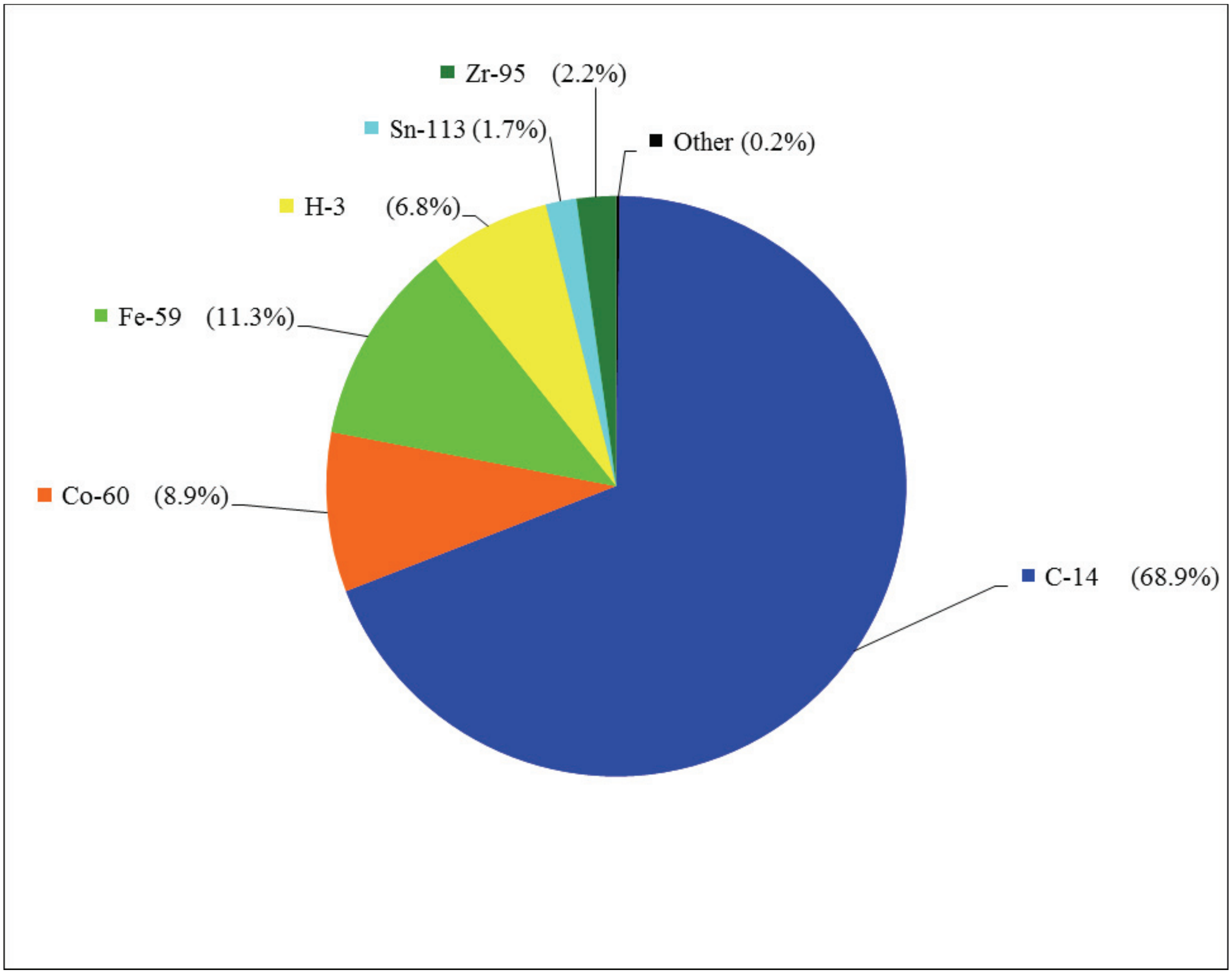


Figure 6.02: Contribution of Radionuclide to Total Dose (Liquid Pathway) – 2015

7 Quality Assurance Results (REMP)

The purpose of Quality Assurance is to provide confidence in the program and demonstrate that the program is able to meet its objectives. QA is a system whereby the laboratory can assure the regulator and NB Power that the laboratory is generating accurate and reproducible data. It encompasses:

- personnel
- procedures
- measurements
- sample integrity
- records
- annual review
- program audits
- program improvement

This section describes how QA was achieved for the year 2015. The specific procedures can be found in *HPF-03541-EN04, Quality Assurance of the Environmental Program.*

7.01 Quality Control Checks

The six main pieces of analytical equipment used in the OERMP have a quality control (QC) check performed at the start of each working day. A background count is made each weekend to ensure the absence of contamination in the gamma spectrometer sample chamber. Key instrument parameters are checked and the results are compared against tolerance limits, and are also compared with previous results to detect trends in performance. This ensures that the parameters are consistent and remain free from significant drift or random variation that could influence the analyses. A compilation of the results and statistical fluctuations is maintained, and from these data the upper and lower flag limits are determined. If any equipment exceeds these limits, it is not used for analytical work until the problem has been resolved. To perform the quality control checks, radiation sources traceable to US or Canadian standards (National Institute of Standards and Technology and National Research Council) are used.

The QC evaluations in the laboratory cover the following instruments:

1. Canberra Intrinsic Ge Gamma Spectrometer
2. Beckman LS 6000TA Liquid Scintillation Counter
3. Tennelec LB-5100 Gross Alpha/Beta Counter
4. Protean WPC 9550 Alpha/Beta Counter
5. Panasonic UD-716AGL TLD Reader
6. Panasonic UD-7900 TLD Reader

Throughout the year there were some results outside expectations for each of the instruments (Table 7.01). Most of these failures involved only one of the six to ten parameters monitored for each system. All of these failures were resolved before analytical work resumed. The unusual number of Tennelec LB-5100 Gross Alpha/Beta Counter failures was due to too tight pass/fail criteria.

Table 7.01: QC Passes & Failures

Instrument	*Number of Parameters Monitored Per Check*	*Number of Checks*	*Number of Individual Parameters Tested*	*Number of Individual Parameters Failed*
Canberra Intrinsic Ge Gamma Spectrometer	6	253	1518	12
Canberra Intrinsic Ge Gamma Spectrometer (Weekend Long Background)	8	54	432	16
Beckman LS 6000TA Liquid Scintillation Counter	10	246	2460	48
Tennelec LB-5100 Gross Alpha/Beta Counter	8	251	2008	139
Protean WPC 9550 Alpha/Beta Counter	8	246	1968	10
Panasonic UD-716AGL TLD Reader	8	228	1824	13
Panasonic UD-7900 TLD Reader	8	236	1888	20

7.01.01 Intrinsic Ge Gamma Spectrometer

A daily check of seven system parameters is performed for the germanium gamma spectroscopy system. Measurements are made of the energy centroids, full width half maxima (FWHM) and efficiencies of two widely separated photon energies of Eu-152. These show the accuracy and precision of the system relative to the defined limits of acceptance. The rate of liquid nitrogen consumption is monitored to verify the physical integrity of the cryostat (this parameter is not reflected in the numbers in Table 7.01). A computer program processes the results to generate QC plots and performs statistical tests to detect out-of-range values. A 200 000 s background count is made each weekend to ensure the absence of contamination in the sample chamber. The QC program evaluates the total counts in eight separate regions of the background spectrum, and out-of-range values are flagged for assessment.

The efficiency calibration of the gamma spectroscopy system is checked annually for each of the counting geometries. This is accomplished using calibration standards derived from a mixed nuclide standard traceable to the U.S. National Institute of Standards and Technology (NIST).

7.01.02 Beckman LS 6000TA Liquid Scintillation Counter

A set of sealed tritium, C-14 and background standards traceable to NIST is analysed daily. Statistical parameters must lie within defined limits or the equipment will not be used. These same standards are used to calibrate the instrument for each analysis run.

7.01.03 Tennelec LB-5100 Gross Alpha/Beta Counter

Planchet standards of Am-241 and Sr-Y-90 are analysed daily. Alpha and Beta discrimination allows the simultaneous analysis of alpha and beta emissions on all samples analysed. Planchet and filter backgrounds are included in the QC checks. These same standards are used to calibrate the instrument for each analysis run.

7.01.04 Protean WPC 9550 Alpha/Beta Counter

Planchet standards of Am-241, Tc-99 and Sr-Y-90 are analysed daily. Alpha and Beta discrimination allows the simultaneous analysis of alpha and beta emissions on all samples analysed. Planchet backgrounds are included in the QC checks. The Tennelec standards are used to calibrate the instrument for each analysis run.

7.01.05 Panasonic UD-716AGL and UD-7900U TLD Readers

In each of the two TLD readers, a set of 16 TLDs is exposed in the Panarad Irradiator and read out in the TLD reader. The mean of each of the four elements, dark current, reference light, reference element, and lamp flashes must all be within specified limits. The QA aspect of this system is covered in detail in the TLD procedures:

- *HPF-03541-TL03, Performing a Quality Control Check on Panasonic Automatic TLD Readers.*
- *HPF-03541-TL09, Performing Quality Assurance Testing of the Dosimetry System.*
- *HPF-03541-TL13, Processing Internal Quality Assurance Test Data*.

7.01.06 Other Instruments

Other instruments (balances, pipettors) are checked or calibrated at least annually. See *HPF-03541-EN05, Calibration, Maintenance and Repair of Equipment Used for the Environmental Program,*. Frequencies of calibration are based on reproducibility of measurements and on time stability tests to ensure that the measurements are within the specified tolerances for accuracy.

The gamma survey and contamination meters are calibrated at PLNGS on an annual basis.

7.02 External QA

The external quality assurance program consists of inter-comparisons with other laboratories to give independent verification of analytical performance. The frequency of each program may vary at the discretion of the sponsoring agency (see Table 7.03). Four such groups – Kinectrics, Eckert & Ziegler Analytics, Environmental Resource Associates (ERA) and the National Research Council (NRC) - provide five percent of the sample load in the laboratory with blind samples. Results of our performance with these samples give an indication of the quality of measurements the laboratory is capable of producing.

The same results are tabulated by medium in Tables 7.04 to 7.11.

The QA agent defines acceptable performance, generally in terms of an expected range. A results outside expectations signals the need to assess the procedures, analytical methods, or equipment calibrations.

There were 11 results that were outside expectations out of 231 nuclide comparisons on 63 samples in the external QA program. The reasons are given in Table 7.02.

Table 7.02: External Quality Assurance Results Outside Expected Range

Analysis Type	***Nuclide***	***Reason***
Analytics Filter	Beta	Could not be determined
COG Filter	Beta	Could not be determined
Analytics Filter	Cr-51	Large uncertainty in reported activity. Suspected detector stability problem.
Analytics Filter	Sr -89	Could not be determined
Analytics Water	Cr-51	Suspected detector stability problem.
Analytics Water	I-131	Suspected detector stability problem.
Analytics Water	Sr-90	Large uncertainty in reported activity. Could not be determined.
COG Water	H-3	Uncertainty in reported activity. Could not be determined.
COG Water	Cd-109	Pass/Fail limits of ±10% compared to all others who use ±3s. At ±15% this would be a pass.
COG C-14 Low	C-14	Pass/Fail limits of ±10% compared to all others who use ±3s. At ±15% this would be a pass.
COG C-14 High	C-14	Uncertainty in reported activity. Could not be determined.

Table 7.03: External Quality Assurance Frequency

Media	*Analyses*	*Number of QA Samples*	*External Agencies*
Filters	Gross Alpha/Beta	2	ERA
		2	Eckert & Ziegler Analytics
		4 (2 gross beta only, 2 gross alpha only)	Kinectrics
	Gamma	2	ERA
		2	Eckert & Ziegler Analytics
	Sr-89,90	2 (only Sr-90, on gamma sample)	ERA
		4	Eckert & Ziegler Analytics
Charcoal Cartridges	Gamma	4	Eckert & Ziegler Analytics
Environmental Gamma	TLD	5	NRC
Milk	Gamma	4	Eckert & Ziegler Analytics
Water	Gross Alpha/Beta	2	ERA
		2	Eckert & Ziegler Analytics
		2 (gross beta only)	Kinectrics
	H-3	4	Kinectrics
	C-14	4	Kinectrics
	Gamma	2	Kinectrics
		4	Eckert & Ziegler Analytics
	Sr-89,90	4 (on gamma sample)	Eckert & Ziegler Analytics
Food/Vegetation	Gamma	2	ERA
		2	Eckert & Ziegler Analytics
Soil/Sediment	Gamma	2	ERA
		2	Eckert & Ziegler Analytics

Table 7.04: Filter Performance (External QA)

Analysis	*QA Agent (pCi·filter^{-1} ± 2 sigma)*	*NB Power (pCi·filter^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
ALPHA	121±4.04	121±6.66	1.00
	46.4±1.5	50.6±3.14	1.09
	62.2	66.1±2.48	1.10
	77.3	79.4±2.85	1.03
	1860±372	1990	1.07
	553±110.6	616	1.11
BETA	241±8.06	294±14.18	1.22
	246±8.22	329±15.58	1.34 Fail
	58.4	71.1±2.47	1.22
	41.3	56.2±2.08	1.36
	801±160.2	926	1.16
	552±110.4	734	1.33 Fail
Am-241	49.8	58.6±8.49	1.18
	36.8	35.5±6.89	0.96
Ce-141	75.8±2.54	76.3±5.9	1.01
Co-58	57.3±1.914	57.7±5.36	1.01
	64.9±2.16	63.3±5.98	0.98
Co-60	162±5.4	158±8.54	0.98
	126±4.2	127±7.2	1.01
	79.1	80.8±4.61	1.02
Cr-51	232±7.74	201±20	0.87
	166±5.54	2.35±0.19	0.01 Fail
Cs-134	137±4.56	105±12.08	0.77
	94.3±3.16	77.8±10.98	0.82
	909	582±34.9	0.64
	349	231±20.7	0.66
Cs-137	105±3.52	101±7.64	0.96
	67.5±2.26	69.2±5.66	1.03
	1170	1050±61.4	0.90
	613	522±31.3	0.85
Fe-59	127±4.24	127±12.22	1.00
	63.4±2.12	72.2±11.74	1.14
Mn-54	84.3±2.82	88±7.04	1.04
	85.2±2.84	92.7±7.42	1.09
Sr-89	154±5.16	146±10.96	0.95
	89.4±2.98	97.4±7.72	1.09
	93.5±3.12	91.5±7.38	0.98
	91.3±3.06	122±9.5	1.34 Fail

Table 7.05 (continued): Filter Performance (External QA)

Analysis	*QA Agent (pCi·filter^{-1} ± 2 sigma)*	*NB Power (pCi·filter^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Sr-90	21±0.702	18.4±2.02	0.87
	12.3±0.4	11.4±1.4	0.93
	15.4±0.516	13.9±1.68	0.90
	13.2±0.44	13.9±1.7	1.05
	96.6	90.6±1.99	0.94
	45.7	45.7±1.48	1.00
Zn-65	208±6.96	217±17.74	1.04
	146±4.88	174±15.32	1.19
	986	1080±375	1.09
	685	705±45.7	1.03

Table 7.06: Charcoal Cartridge Performance (External QA)

Analysis	*QA Agent (pCi·cartridge^{-1} ± 2 sigma)*	*NB Power (pCi·cartridge^{-1} ± 2 sigma)*	*NB Power/QA Agent (ratio)*
I-131	85.6 ± 2.86	98.1 ± 13.98	1.15
	80±2.68	92.4±21.8	1.15
	81.7±2.72	69.4±8.54	0.85
	75.3±2.52	55.3±7.28	0.73

Table 7.07: Milk Performance (External QA)

Analysis	*QA Agent (pCi·L^{-1} ± 2 sigma)*	*NB Power (pCi·L^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Ce-141	4.40 ± 0.14	3.65 ± 0.27	0.83
	4.59 ± 0.15	4.29 ± 0.33	0.94
	8.10 ± 0.27	8.18 ± 0.54	1.01
Co-58	9.92 ± 0.33	9.29 ± 0.62	0.94
	4.14 ± 0.14	3.81 ± 0.30	0.92
	5.29 ± 0.17	5.18 ± 0.36	0.98
	4.81 ± 0.16	4.59 ± 0.33	0.95

Table 7.07 (continued): Milk Performance (External QA)

Analysis	*QA Agent (pCi·L^{-1} ± 2 sigma)*	*NB Power (pCi·L^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Co-60	405±13.4	429±23	1.06
	193±6.44	206±11.58	1.07
	330±11.02	347±18.76	1.05
	213±7.12	230±12.8	1.08
Cr-51	276±9.2	274±30	0.99
	276±9.22	281±29	1.02
	538±17.98	520±49	0.97
	281±9.38	280±34.4	1.00
Cs-134	154±5.14	163±16.68	1.06
	163±5.44	160±15.7	0.98
	212±7.08	213±18.82	1.01
	160±5.36	170±16.84	1.06
Cs-137	207±6.9	212±15.54	1.03
	125±4.18	129±9.96	1.03
	255±8.52	255±18.42	1.00
	115±3.82	117±9.08	1.02
Fe-59	177±5.92	201±16.9	1.14
	151±5.06	169±14.54	1.12
	226±7.54	245±20.6	1.09
	108±3.6	122±13.96	1.13
I-131	65.1±2.18	63.9±6.8	0.98
	95.9±3.2	97±8.6	1.01
	99.9±3.34	101±11.04	1.01
	91.2±3.04	104±17.72	1.14
Mn-54	188±6.28	203±15.08	1.08
	101±3.36	108±8.62	1.07
	290±9.7	309±22.2	1.06
	145±4.84	158±12	1.09
Zn-65	351±11.72	383±29.2	1.09
	248±8.3	284±22.4	1.14
	353±11.8	375±28.6	1.06
	248±8.28	270±21.6	1.09

Table 7.08: Water Performance (External QA)

Analysis	*QA Agent ($pCi{\cdot}L^{-1}$ ± 2 sigma) or ($pCi{\cdot}kg^{-1}$ ± 2 sigma)*	*NB Power ($pCi{\cdot}L^{-1}$ ± 2 sigma) or ($pCi{\cdot}kg^{-1}$ ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
ALPHA	125±4.16	162±13.58	1.3
	46.4±1.5	42.7±4.16	0.92
	119	131±12.1	1.10
	136	165±14.3	1.21
BETA	248±8.3	264±18.58	1.06
	247±8.24	278±19.6	1.13
	158	141±10.4	0.90
	53.7	70.5±5.96	1.31
Am-241	2.3±0.23	2.53	1.1
	2.14±0.214	2.32	0.18
C-14	12±1.2	11.6	0.97
	160±16	151	0.94
	0.30±0.03	0.26	0.87 Fail
	13.9±1.39	9.48	0.68 Fail
Cd-109	14.7±1.47	15.9	1.08
	27.4±2.74	30.6	1.12 Fail
Ce-139	0.25±0.025	0.26	1.04
	0.98±0.098	0.97	0.99
Ce-141	139±4.46	160±20.8	1.15
	199±6.64	202±21	1.02
	112±3.74	129±17.88	1.15
Co-57	0.32±0.032	0.33	1.03
	0.66±0.066	0.68	1.03
Co-58	180±6	168±21.4	0.93
	72.6±2.42	81.9±13.53	1.13
	246±8.2	250±26.6	1.02
	95.6±3.2	113±17.16	1.19
Co-60	328±10.96	357±27	1.09
	205±6.84	204±18.58	1.00
	308±10.28	333±25.6	1.08
	185±6.18	185±17.18	1.00
	1.38±0.14	1.46	1.06
Cr-51	366±12.22	372±90	1.02
	329±66.2	293±9.8	1.12
	502±16.78	504±87	1.00
	244±8.14	341±80.6	1.40 Fail

Analysis	*QA Agent (pCi·L^{-1} ± 2 sigma) or (pCi·kg^{-1} ± 2 sigma)*	*NB Power (pCi·L^{-1} ± 2 sigma) or (pCi·kg^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Cs-134	126±4.2	114±16.46	0.90
	173±5.78	158±27.8	0.91
	198±6.6	183±21.6	0.93
	139±4.64	133±17.36	0.96
Cs-137	167±5.56	175±20.2	1.05
	133±4.44	134±17.06	1.01
	238±7.94	243±25.2	1.02
	99.5±3.32	112±14.9	1.13
	0.60±0.06	0.63	1.05
	0.84±0.08	0.83	0.99
Fe-59	195±6.5	191±44	0.98
	161±5.38	150±32.6	0.93
	211±7.04	250±42	1.19
	93.4±3.12	80.6±28.2	0.86
H-3	9±0.9	11.7	1.3 Fail
	70±7	72.5	1.04
	5.0±0.5	4.94	0.99
	220±22	222	1.01
Hg-203	0.073±0.0073	0.08	1.09
	2.09±0.21	2.02	0.97
I-131	93.4±3.12	93.4±14.34	1.00
	96.7±3.24	100±17.46	1.03
	92.6±3.1	125±44.6	1.35 Fail
Mn-54	159±5.3	178±21	1.12
	107±3.56	107±15.48	1.00
	271±9.04	290±29	1.07
	126±4.2	140±17.86	1.11
Sn-113	0.32±0.032	0.33	1.03
	1.7±0.17	1.71	1.00
Sr-85	2.03±0.2	2.08	1.02
Sr-89	99.2±3.32	116±8.46	1.17
	97.7±3.26	104±7.9	1.06
	91±3.04	110±8.26	1.21
	89.7±3	94.7±7.38	1.06

Table 7.08 (continued): Water Performance (External QA)

Analysis	*QA Agent (pCi·L^{-1} ± 2 sigma) or (pCi·kg^{-1} ± 2 sigma)*	*NB Power (pCi·L^{-1} ± 2 sigma) or (pCi·kg^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Sr-90	13.5±0.452	21.8±2.26	1.61 Fail
	13.4±0.446	16.8±2.24	1.26
	15±0.5	15.1±2.02	1.00
	13±0.43	15.6±2.1	1.20
Y-88	0.66±0.066	0.69	1.04
	2.86±0.29	3.04	1.06
Zn-65	299±9.98	318±43.2	1.06
	264±8.82	291±41	1.10
	330±11	359±46.4	1.09
	215±7.2	212±34.6	0.98

Table 7.09: Food/Vegetation Performance (External QA)

Analysis	*QA Agent (pCi·kg^{-1} ± 2*	*NB Power (pCi·kg^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Am-241	4.34	5.61±0.503	1.29
	1.59	1.98±0.25	1.24
Ce-141	0.224±0.007	0.28±0.04	1.25
	0.117±0.004	0.122±0.024	1.04
Co-58	0.192±0.006	0.23±.0.4	1.20
	0.331±0.01	0.34±0.032	1.03
Co-60	0.371±0.012	0.46±0.04	1.24
	1.54	1.71±0.125	1.11
	1.93	1.85±0.13	0.96
	0.489±0.016	0.56±0.16	1.14
Cr-51	0.279±0.009	0.273±0.034	0.98
Cs-134	0.279±0.009	0.32±0.02	1.15
	2.65	2.71±0.289	1.02
	0.75	0.71±0.09	0.95
	4.34	5.61±0.503	1.29
Cs-137	0.215±0.007	0.207±0.028	0.96
	0.2±0.006	0.24±0.02	1.20
	1.81	2.09±0.179	1.15
	1.23	1.19±0.12	0.97
Fe-59	0.26±0.009	0.264±0.068	1.02
	0.187±0.006	0.25±0.06	1.33
K-40	30.9	33.8±2.86	1.09
	31	31.4±2.68	1.01
Mn-54	0.173±0.006	0.181±0.028	1.05
	0.252±0.008	0.32±0.04	1.27

Table 7.09 (continued): Food/Vegetation Performance (External QA)

Analysis	*QA Agent (pCi·kg^{-1} ± 2 sigma)*	*NB Power (pCi·kg^{-1} ± 2 sigma)*	*NB Power/QA Agent (ratio)*
Zn-65	0.427±0.0142	0.455±0.07	1.07
	0.432±0.014	0.54±0.08	1.25
	1.09	1.22±0.185	1.12
	1.54	1.68±0.22	1.09

Table 7.10: Soil Performance (External QA)

Analysis	*QA Agent (pCi·kg^{-1} ± 2 sigma)*	*NB Power (pCi·kg^{-1} ± 2 sigma)*	*NB Power/ QA Agent (ratio)*
Am-241	1.5	1.28±0.164	0.85
Ce-141	0.222±0.01	0.022±.04	0.99
Co-58	0.119±0.004	0.122±0.026	1.02
	0.19±0.006	0.16±0.04	0.84
Co-60	0.336±0.0112	0.347±0.032	1.03
	0.368±0.012	0.34±0.04	0.92
	1.88	1.89±0.098	1.00
	3.9	3.76±0.18	0.96
Cs-134	0.284±.01	0.285±.036	1.00
	0.277±0.01	0.23±0.02	0.83
	6.39	5.38±0.357	0.84
	2.42	2.15±0.21	0.89
Cs-137	0.298±0.01	0.271±0.034	0.91
	0.276±0.01	0.25±0.04	0.91
	1.49	1.37±0.103	0.92
	5.12	4.69±0.3	0.92
Fe-59	0.264±.0088	0.277±0.076	1.05
	0.186±0.006	0.14±0.06	0.75
K-40	10.7	10.6±0.89	0.99
	10.6	10.6±0.88	1.00
Mn-54	0.176±0.006	0.191±0.028	1.09
	0.25±0.01	0.25±0.04	1.00
Zn-65	0.434±.0145	0.474±0.052	1.09
	0.429±0.014	0.4±0.04	0.93
	7.13	7.27±0.462	1.02
	3.62	3.83±0.256	1.06

Table 7.11: Environmental TLD Performance (External QA)

Analysis	*QA Agent (mR ± 2 sigma)*	*NB Power (mR ± 2 sigma)*	*NB Power/QA Agent (ratio)*
Gamma	116 ± 6	106 ± 17	0.91

7.03 Internal QA

There are three parts to Internal QA:

1) duplicate samples – two samples collected at the same time and analysed separately
2) replicate analyses – two analyses done on the same sample
3) blind analyses – one person irradiates the TLDs and a different person performs the analysis

Duplicate samples and replicate analyses are employed as part of the overall quality assurance program. For those media where two samples can be obtained from the same location at the same time, similar analytical results are expected. This approach demonstrates that the samples are representative of the medium in that area. Where duplicate samples are not possible, e.g., air filters, a sample is counted twice to demonstrate reproducibility in the counting system. Tracking of results is done in a spreadsheet and performance is charted. If the range of the ratio (of the two detected measurements) plus or minus the combined uncertainty (95% confidence interval) includes 1.00, then performance is acceptable. See Table 7.13 for the frequency.

There were 173 radionuclide comparisons performed on 84 samples analysed. Twelve of these had results outside expectations in one of the comparisons.

The results are presented graphically in Figures 7.01 to 7.12 (plotted against the analysis date).

Table 7.13: Internal Quality Assurance Frequency

Medium	*Duplicate/Replicate*	*Number of Radionuclide Comparisons*	*Analyses*
Airborne Carbon Dioxide	Replicate analysis (single location)	12	LSC C-14
Airborne Iodines	Replicate count (1 composite set)	22	Gamma
Airborne Particulates	Replicate analysis	12	Gamma
		24	Alpha/Beta
Food	Replicate analysis	8	Gamma
Milk	Duplicate sample	4	Gamma
Parshall Flume	Replicate analysis	10	LSC H-3
LEM Composite	Replicate analysis	25	Gamma
		23	Alpha/Beta
		8	Sr-89,90
Sea Food	Replicate analysis	1	Gamma
Sediment / Soil	Duplicate sample	16	Gamma
Environmental Gamma	Duplicate sample	4	TLD

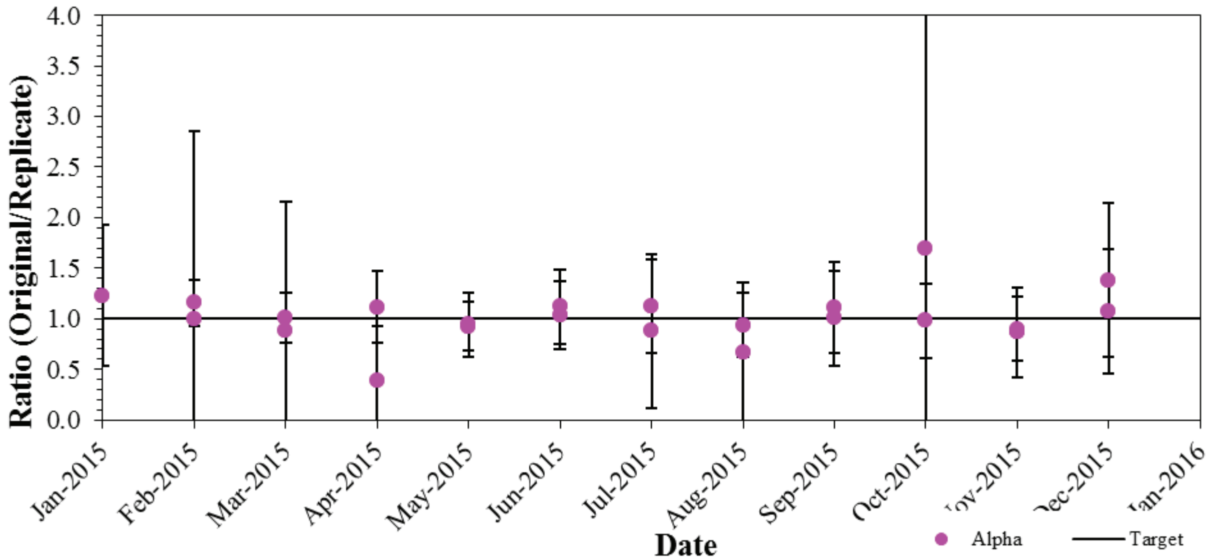


Figure 7.01: Alpha Performance (Internal QA – duplicate/replicate)

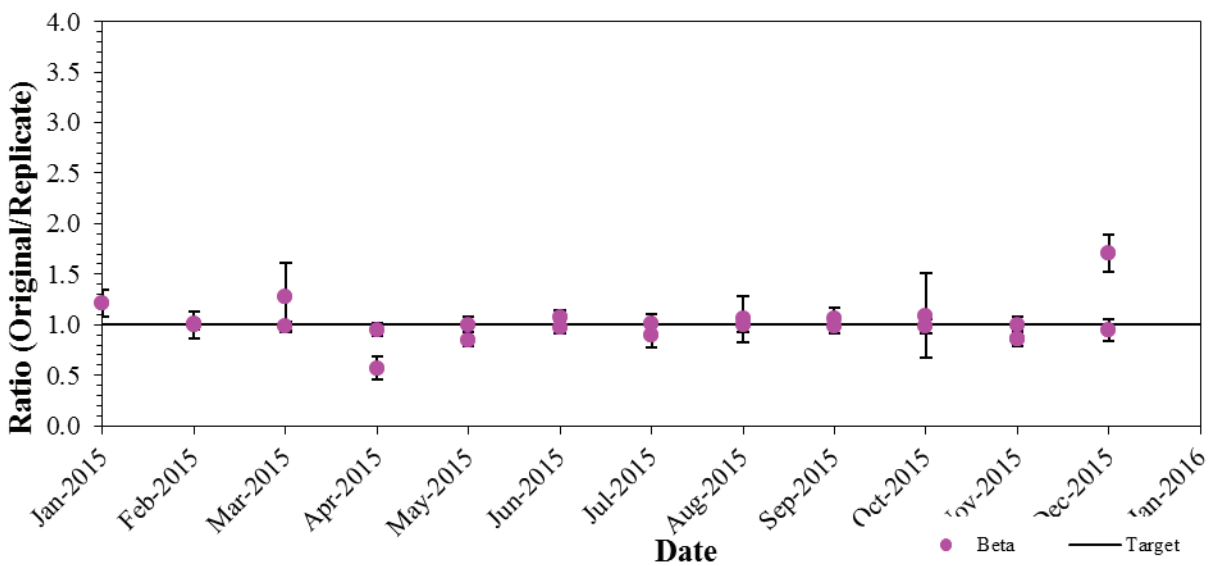


Figure 7.02: Beta Performance (Internal QA – duplicate/replicate)

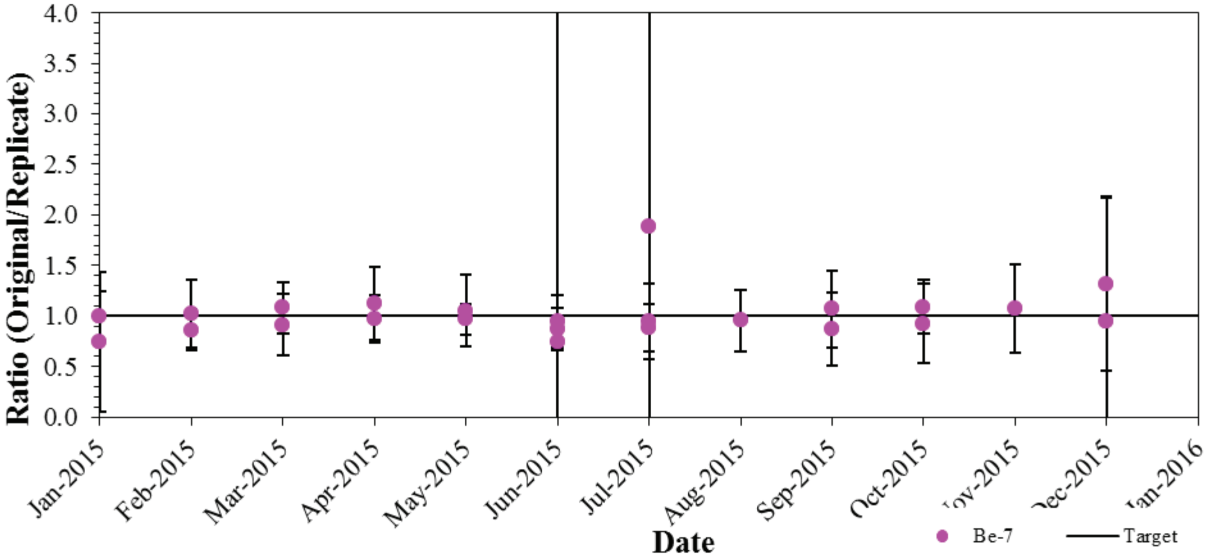


Figure 7.03: Beryllium-7 Performance (Internal QA – duplicate/replicate)

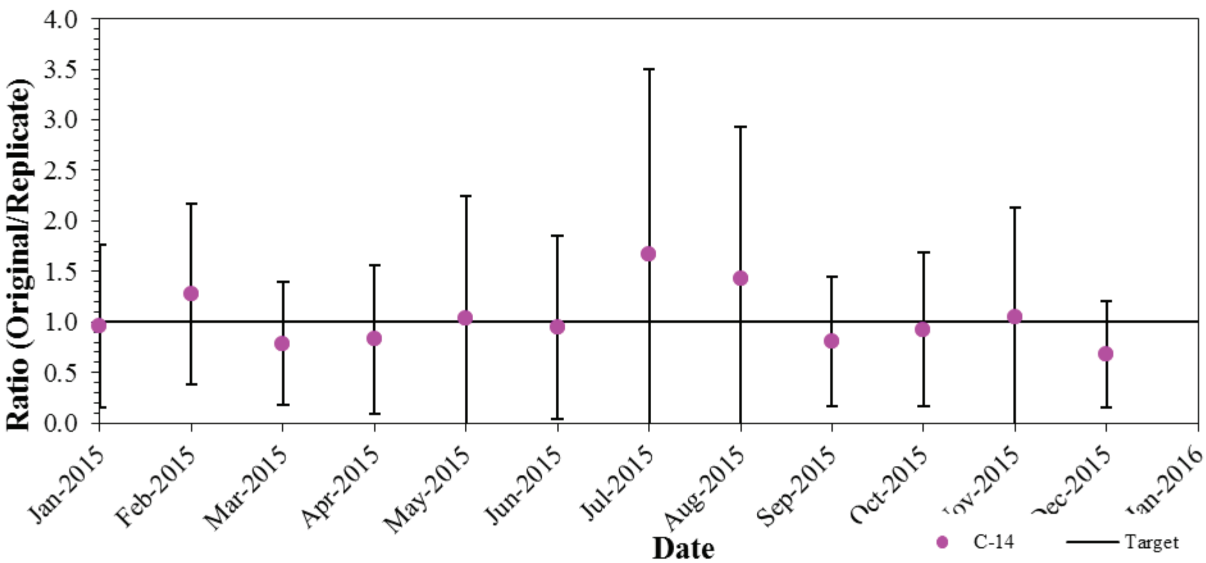


Figure 7.04: Carbon-14 Performance (Internal QA – duplicate/replicate)

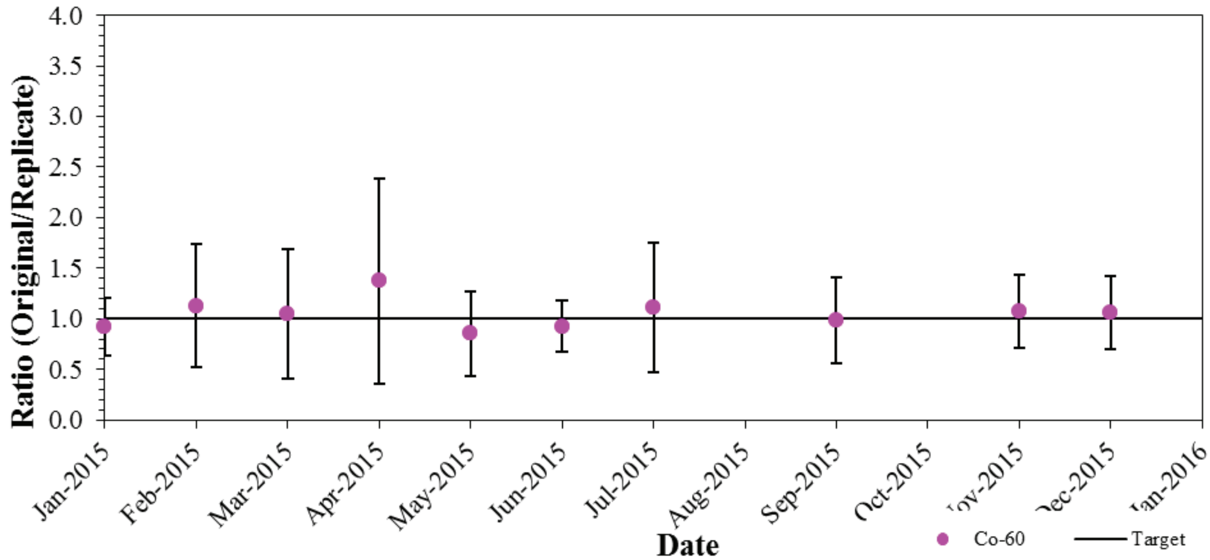


Figure 7.05: Cobalt-60 Performance (Internal QA – duplicate/replicate)

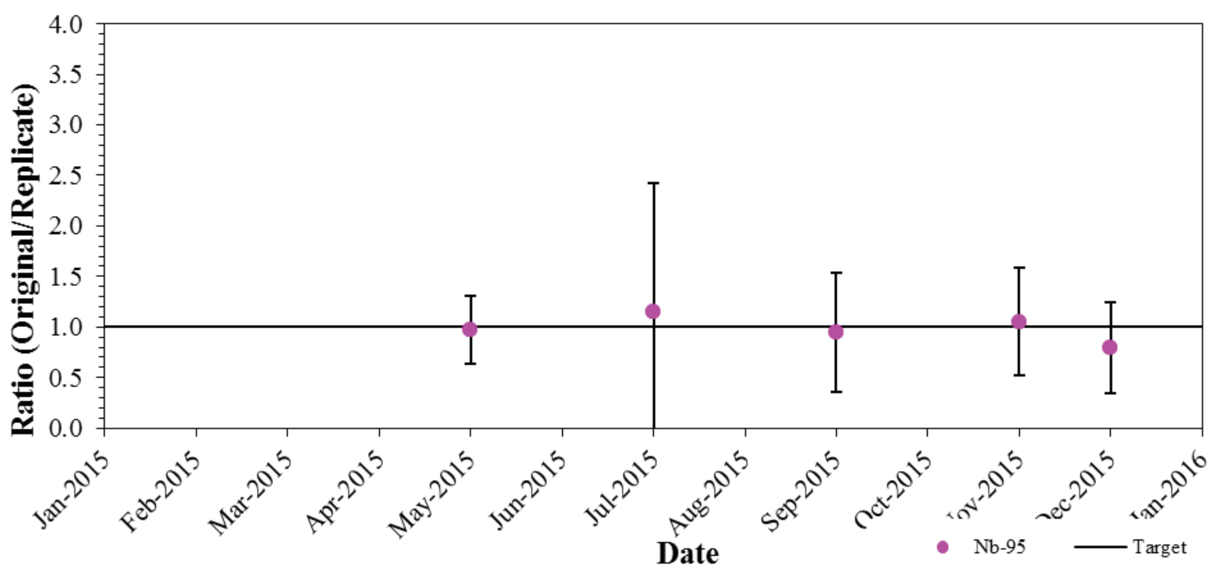


Figure 7.06: Niobium-95 Performance (Internal QA – duplicate/replicate)

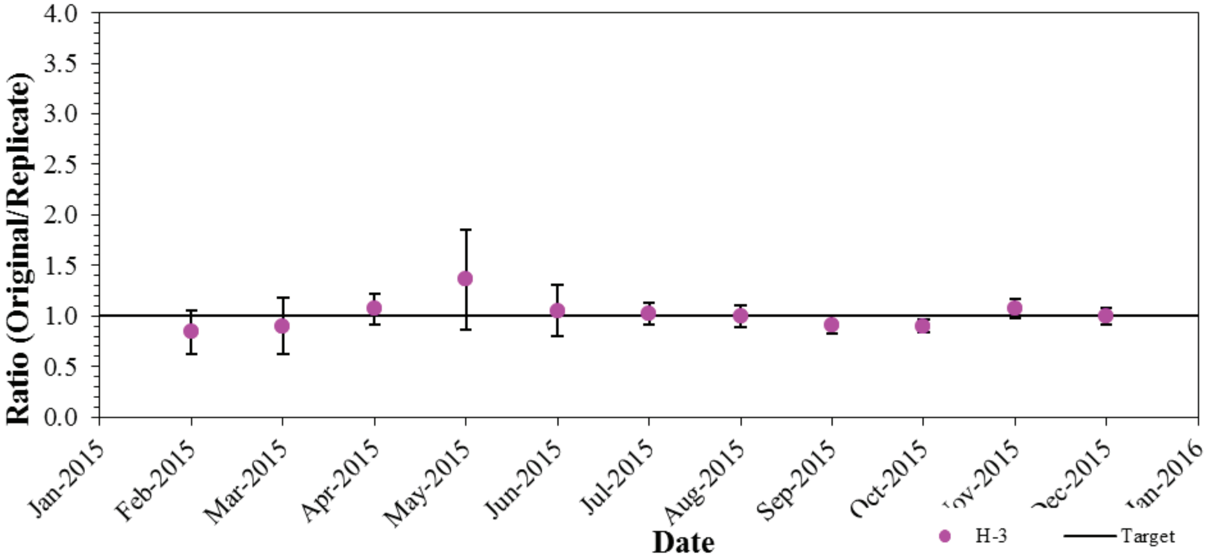


Figure 7.07: Tritium Performance (Internal QA – duplicate/replicate)

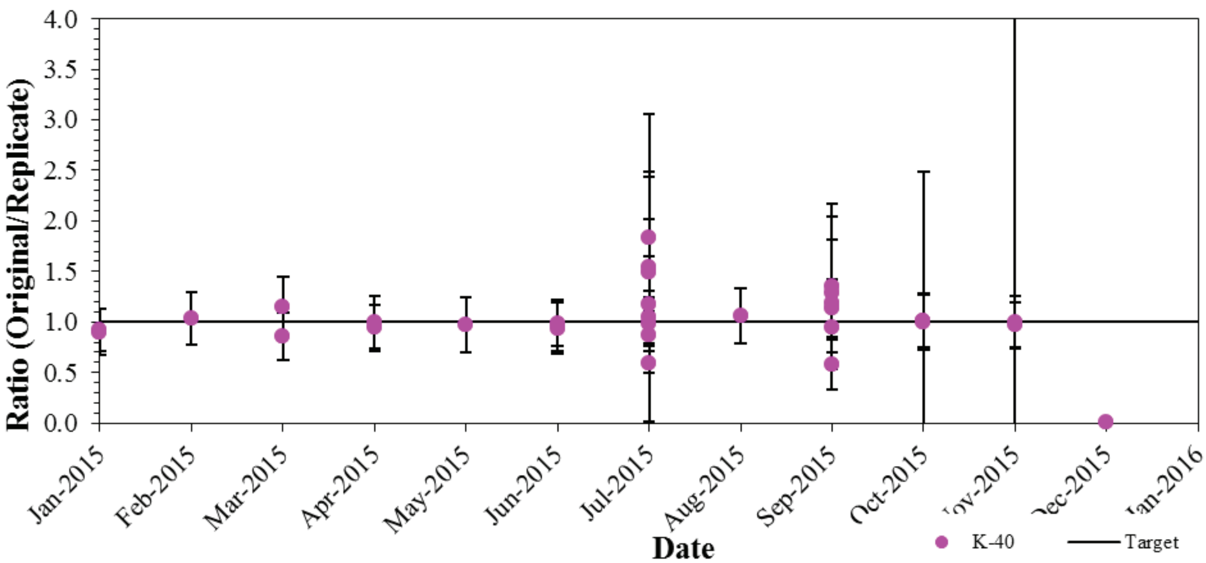


Figure 7.08: Potassium-40 Performance (Internal QA – duplicate/replicate)

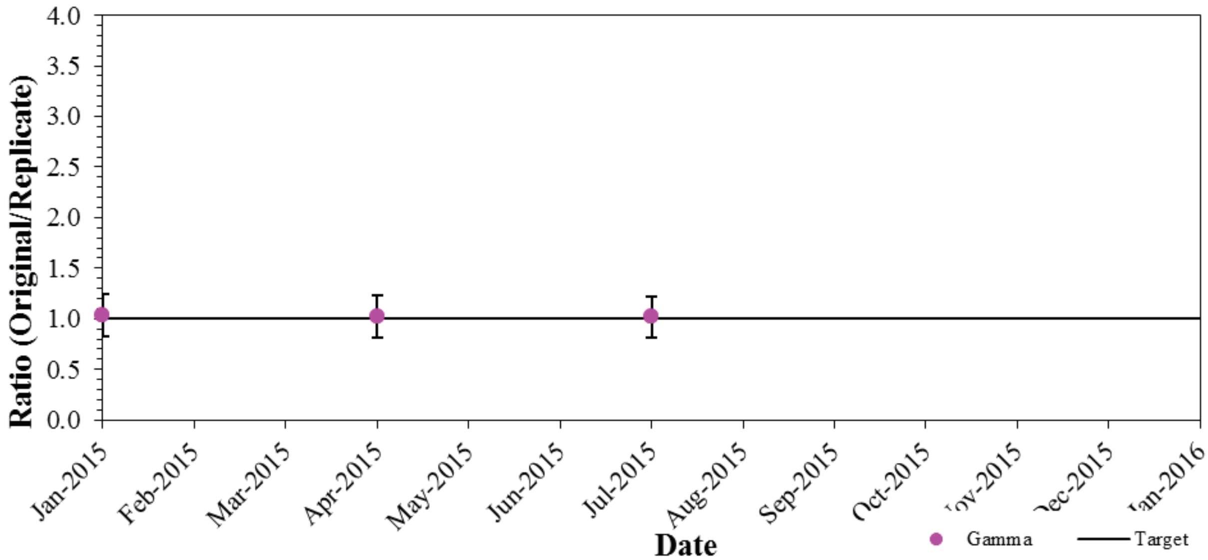


Figure 7.09: Gamma Performance (Internal QA – duplicate/replicate)

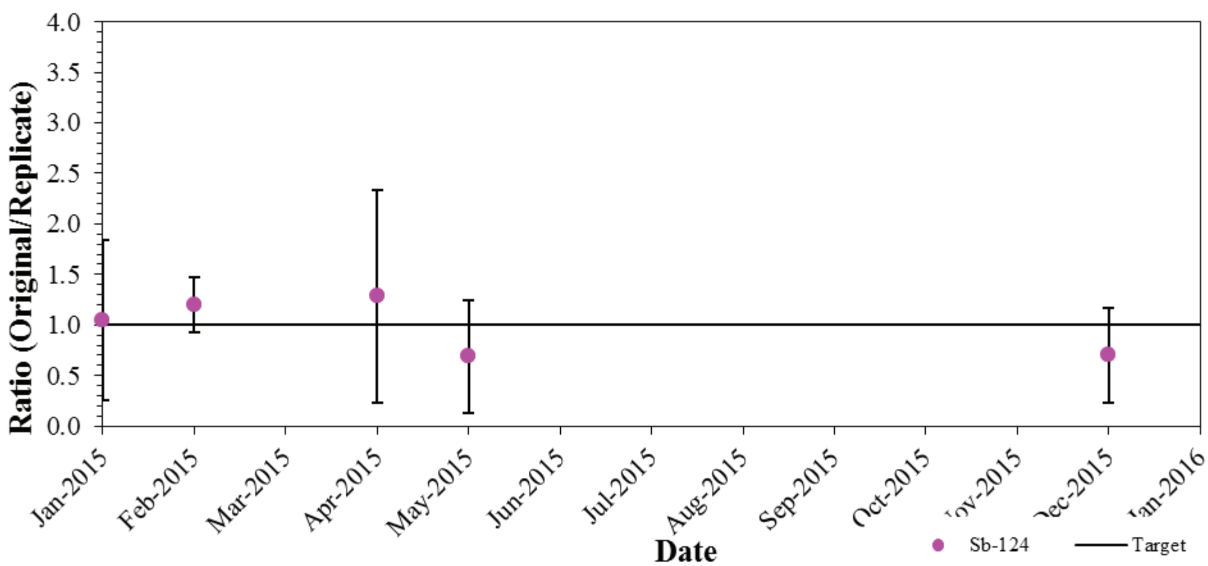


Figure 7.10: Sb-124 Performance (Internal QA – duplicate/replicate)

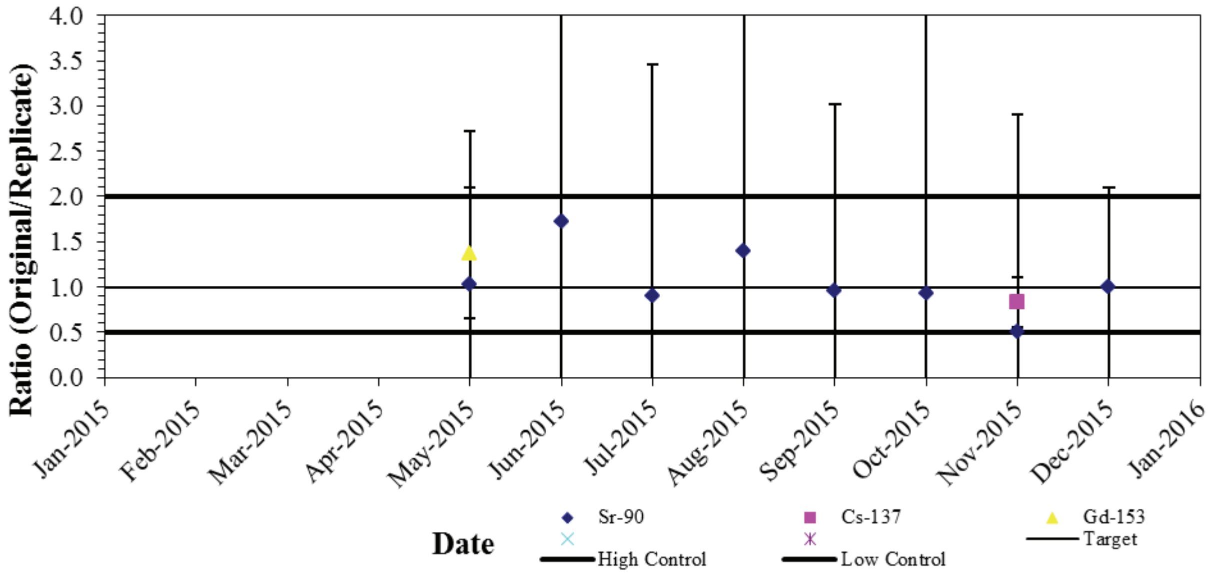


Figure 7.11: Strontium-90, Cs-137 and Gd-153 Performance (Internal QA – duplicate/replicate)

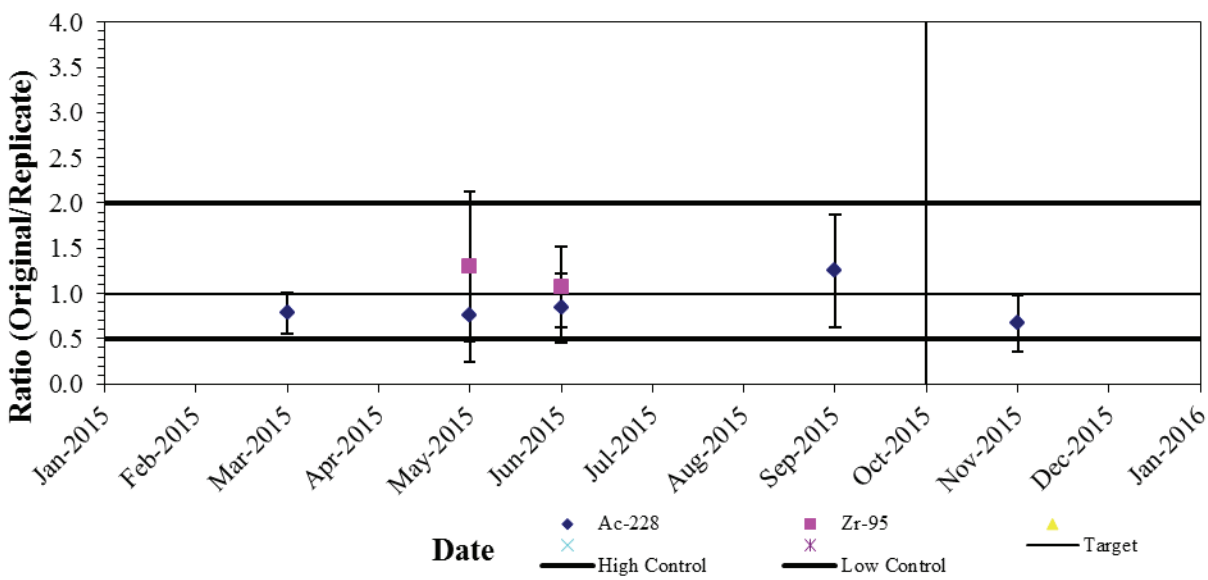


Figure 7.12: Actinium-228 and Zr-95 Performance (Internal QA – duplicate/replicate)

Samples that are spiked by laboratory personnel play a minor role in the QA program. It is more desirable to purchase QA samples from an accredited QA laboratory. The only exception is the irradiation of environmental TLDs. Similar to External QA, one member of the lab staff irradiates the TLDs and a different person analyses the sample. Results of performance with these samples give an indication of the quality of measurements. Acceptable performance is defined as results within ± 15% of the expected value.

The four separate tests were successful (five TLDs for each test). The results are presented in Figure 7.13.

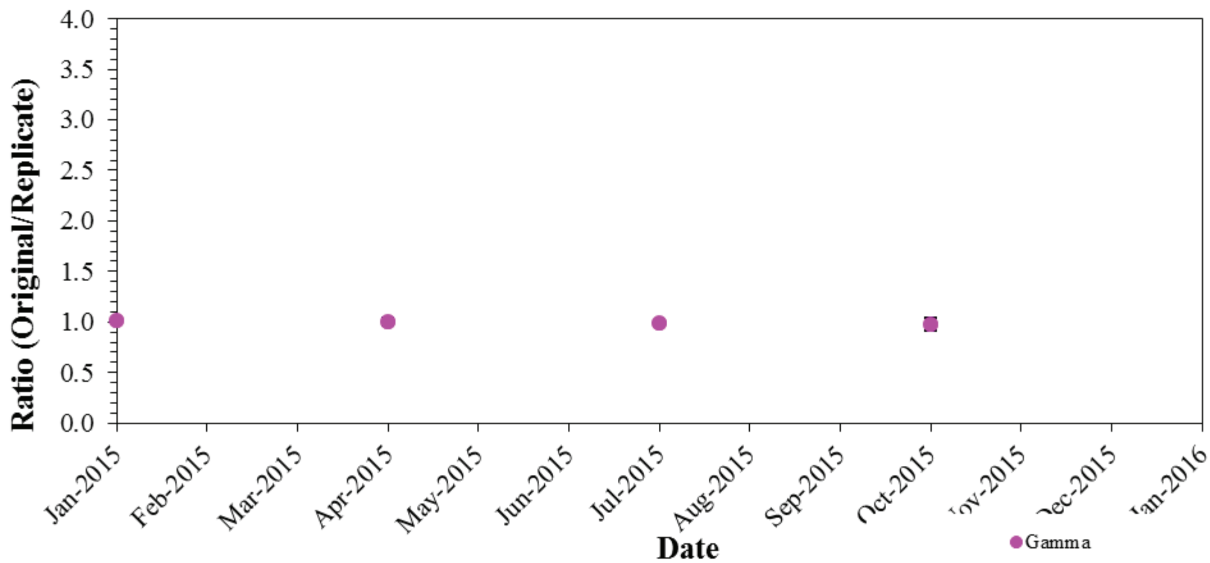


Figure 7.13: Gamma Performance (Internal QA - spikes)

7.04 Program Audit

The REMP audit frequency was changed to once every five years to align with the Canadian Standards Association (CSA) standard. The Nuclear Oversite Group (NOS) at PLNGS is the principal auditor, although other groups from within NB Power, the CNSC, or other utilities may be used.

As part of its overall Management System, Point Lepreau has an Environmental Management System (EMS) in place that is registered to ISO 14001. Radiological releases to water and air are part of this system. There were two audits relating to the EMS during 2015.

7.05 Annual Review

A review of the Radiation Environmental Monitoring Program (REMP) takes place each year. The requirement for this review is stated in *IR-03541-HF02, Radiation Environmental Monitoring Program (REMP)* and described in *HPF-03541-EN04, Quality Assurance of the Environmental Program*. All members of the lab take part in this formal review. Generally, all aspects of the program are examined to make sure that the objectives, as stated in the program description document, are being met.

8 Non-Radiological Monitoring and Reporting

8.01 Ozone Depleting Substance

In Canada, the federal, provincial and territorial governments have legislation in place for the protection of the ozone layer and management of ozone-depleting substances and their halocarbon alternatives. The use and handling of these substances are regulated by the provinces and territories in their respective jurisdictions, and through the Federal Halocarbon Regulation, 2003 for refrigeration, air-conditioning, fire extinguishing, and solvent systems under federal jurisdiction. Point Lepreau Nuclear Generating Station is governed by both the provincial and federal regulations.

In 2015, a release of 13.2 kg of R-22 was reported to Environment Canada.

Letters submitted to Environment Canada are sent to the CNSC staff as per *Guidance* in REGDOC 3.1.1 Section 3.5.

8.02 Domestic Waste Water Treatment (Sewage) (Approval to Operate S-2696)

The domestic waste water is regulated by the provinces and territories in their jurisdictions, and through the Federal Wastewater System Effluent Regulations. Point Lepreau Nuclear Generating Station is governed federally and administered provincially.

At Point Lepreau Nuclear Generating Station, an electronic report via Effluent Regulation Reporting Information System (ERRIS) is completed. The electronic submission frequency is determined on daily discharge flow of the facility. PLNGS electronic report was completed January 14, 2016. Based on the 2014 daily discharge flow, it was required to be completed electronically once during the year at the end of the calendar year. Also, a letter is submitted to New Brunswick Department of Environment and Local Government describing any discharge to an Overflow Points and Environmental Emergency that occurred during the year. This was submitted on January 15, 2016. The letter is required to be submitted within 45 days of the end of each year.

The approval required to sample (grab or composite) on a quarterly basis but at least 60 days after any other samples. PLNGS collect and analyses the effluent on a weekly basis to verify the performance of the facility.

The samples collection and analysis are performed by Saint John Laboratory Services Ltd. They are accredited to Canadian Association for Laboratory Accreditation Inc. (CALA).

Table 8.01 Electronic Data Submission to ERRIS

Waste Water System Summary	
Owner:	New Brunswick Power Corporation

Wastewater System:	Point Lepreau Generating Station		
Approval State:	Approved		
Reporting year:	2015	**Reporting Period**:	January to December
System Type:	Continuous	**Average Daily Effluent Volume (m³):**	114.7
Reporting Frequency:	Annually	**Averaging Period:**	Annually

Effluent Monitoring Data

Month	Effluent Deposited?
January	Yes
February	Yes
March	Yes
April	Yes
May	Yes
June	Yes
July	Yes
August	Yes
September	Yes
October	Yes
November	Yes
December	Yes

Number of days that effluent was deposited	Total volume of effluent deposited (m³)	Average CBOD (mg/L) Limit: 25 mg/L	Average concentration of suspended solids(mg/L) Limit: 25 mg/L
365	40489.0	1.2	1.3

Acute Lethality Test Results

Does your waste water system have Acute Lethality test sample (s) to report in this reporting period? **(Required)** No

This test is required when daily flow is greater than 2,500 m³.

8.03 Waste Water Compliance (Approval to Operate I-7479)

The monthly wastewater compliance reports for PLNGS are submitted to New Brunswick Department of Environment and Local Government, based on the reporting Conditions of the Approval to Operate, as follows:

The operation of the Industrial Wastewater Treatment System at PLNGS has an Approval to Operate (#I-7479) issued under the Water Quality Regulation – Clean Environment Act. It is valid from May 1, 2011 until April 30, 2016. Condition 43 of the Approval states that "Within 60 days of the end of each month, the Approval Holder shall submit the results of each month's monitoring program to the Director". Condition 44 states that "As part of the December report for each year, The Approval Holder shall submit an Annual Environmental Report to the Department."

Samples are collected and analyzed daily for pH, suspended solids and hydrazine. From the daily samples, a monthly composite is prepared and analyzed for heavy metals (arsenic, barium, cadmium, chromium, copper, iron, lead, mercury, nickel, vanadium and zinc) and Total Petroleum Hydrocarbons (TPH).

The daily sample analysis is performed by the Chemistry Department using procedures:

- CAP-78200-PH1; *pH Measurement by Glass Combination Electrode*
- CLIP-78200-74; *Accumet Excel Model 25 pH/Millivolt Meter*
- CAP-78200-SU2; *Suspended Solid*
- CAP-78200-HY1; *Hydrazine by P-Dimethylaminobenzaldehyde*
- CLIP-78200-22; *Varian Cary 50 UV/VIS Spectrometer*
- CMP-78200-03; *Varian UV/VIS Spectrometer Model Cary*

The heavy metals and the TPH are analysis is performed by Saint John Laboratory Services Ltd. They are accredited to Canadian Association for Laboratory Accreditation Inc. (CALA).

Monthly reports and the annual report are sent to the CNSC staff as per *Guidance* in REGDOC 3.1.1 Section 3.5.

8.04 Air Emission (NPRI)

Site conventional air emissions are controlled to meet regulatory requirements, prevent pollution, reduce emissions, and to minimize environmental impacts.

Point Lepreau Nuclear Generating Station no longer requires an air quality approval to operate the Auxiliary Volcano Boiler and Diesel Generators. The fuel consumption and emissions for 2015 were tracked and calculated for possible reporting purposes to the National Pollutant Release Inventory (NPRI) should emissions meet reporting thresholds. We do not need to report the emissions to the Department of Environment and Local Government.

During the year 1,840 barrels (292,560 liters) of Type 2 Light Oil and 4,678 barrels (743,802 liters) of Type B Diesel Fuel were consumed at the station. The preliminary analysis indicate the light fuel oil had an average energy content of 5.84 million BTUs per barrel, an average ash content of 0.0003 percent, and an average sulphur content of 0.0036 percent. The preliminary analysis indicate the diesel fuel oil had an average energy content of 5.84 million BTUs per barrel, an average ash content of 0.0005 percent, and an average sulphur content of 0.0007 percent. Fuel analysis results are obtained from the AmSpec Services analysis results sent to the Chemistry Department at PLNGS while fuel consumption figures are provided by the NB Power Fuels Group.

During the year the annual emissions were calculated and are shown in Table 8.02

Table 8.02 Annual Emissions (2015)

Parameter	Tonnes
Carbon Dioxide	3,079
Sulphur Dioxide	0.03
Nitrogen Dioxide	8.32
Particulate Matter	0.36

8.05 Chlorine

There is currently no chlorine disinfection on site at the Point Lepreau Nuclear Generating Station. There is a sodium hypochlorite system utilized during maintenance of specific sections of the domestic waste water works.

8.06 Ammonia

There is currently no requirement to measure ammonia in the effluent at Point Lepreau Nuclear Generating Station.

8.07 Hydrazine

Hydrazine is reported with the Inactive Waste Water approval to Operate I-7475. Samples are collected and analyzed daily at the lagoon discharge, Turbine Building sump 8 and the Ditch.

8.08 EMS Program Audit

The Point Lepreau Nuclear Generating Station is certified to ISO 14001:2004 standard. The certification cycle is a period of three years. The 2016 was a maintenance audit year. During the audit, the auditor identified four (4) Minor nonconformities with seven (7) opportunities for improvement and three (3) observations. All findings were minor in nature and are being track through PLNGS's internal Corrective Action Program.

8.09 Self-Assessments

In 2015, the environmental group performed five self-assessments**.**

A) Benchmarking the Environmental Management System (EMS)

There were four benchmarking performed with various companies to improve the EMS program. Based on those benchmarks, PLNGS was able to simplify and improved the ranking of Significant Environmental Aspect, Environmental Database, environmental practices, Management Reviews and training/awareness. The proposed actions were documented as part of continuous improvement.

B) Transportation of Dangerous Good (TDG) reconciliation

The environmental group performed a focused self-assessment to ensure the reconciliation of manifest documents for TDG and actions taken to close a non-conformance of an EMS audit were effective and sustained. A visit in the field and interviews with supply staff as well as personnel from training confirmed actions taken to address the non-conformance that the third party auditor had originally identified in September 2014 has been sustained and effective. In addition, ongoing efforts to improve the process have been observed and dedicated staff in supply and environment is ensuring oversight is maintained in this area. Based on the self-assessment, there were no proposed actions taken.

9 Reports and Studies

A gap analysis for alignment to the *CSA Standard N288.4-10, Environmental monitoring programs at Class I nuclear facilities and uranium mines and mills* was conducted in 2012. Implementation plans were made in 2013. Alignment to the Canadian Standards Association (CSA) standards *N288.4-10, Environmental monitoring programs at Class I nuclear facilities and uranium mines and mills* and *N288.5-11, Effluent monitoring programs at Class I nuclear facilities and uranium mines and mills* was progressed in 2015. The following were in the review stage at the end of 2015:

- Ecological risk assessment report (*Point Lepreau Generating Station- Site Wide Risk Assessment: Human Health and Ecological Risk Assessment, February 2016, Arcadis Canada Inc)* as per *N288.6-12, Environmental risk assessments at class I nuclear facilities and uranium mines and mills*).
- Fish entrainment report (*Point Lepreau Generating Station- Final: Entrainment Monitoring Plan and Implementation for Point Lepreau Generating Station, March 2016, Arcadis Canada Inc.*).
- Fish impingement report (*NB Power- Progress Report Impingement Monitoring at Point Lepreau Generating Station 2013-2014, March 2016, Arcadis Canada Inc*.).

A contract has been awarded to Canadian Nuclear Laboratories (CNL) to close the gaps and implement the standards.

Appendix A: Statistics, Detection Limits, and Dose at Detection Limits

A1 Statistics

The following statistical conventions are applied in the analysis of each sample:

- Detection limits are defined following the method described by Lochamy in *NBS Special Publication 456, Measurements for the Safe Use of Radiation (US Department of Commerce, 1976).* The lower limit of detection (LLD) at the 99% confidence level is defined as 6.58 S_b, where S_b is the standard deviation of the appropriate radiation background measurement. This LLD corresponds to that amount of activity in a sample that will yield a net count greater than 3.29 S_b, or the so-called critical level (CL), with 99% probability. Thus, the LLD specifies the theoretical capability of the system to detect a given amount of radioactivity, whereas the CL is used to determine whether an actual activity measurement should be considered detected. Any net measurement greater than 3.29 S_b is considered detected at the 99% confidence level. This also implies a one percent probability of stating that activity is present when it is not (false positive). If activity is present at the LLD level (6.58 S_b), there is a one percent probability of stating that activity is not present when it is (false negative).

- The CL of 3.29 S_b and LLD of 6.58 S_b apply in those analytical systems where the background levels are either not well defined, or where there is a relationship between the background levels and the detected signal above background, as in Ge gamma spectroscopy. Where the background readings are well defined and are independent of sample readings, as in the TLD data, the CL is 2.33 S_b and the LLD is 4.66 S_b.

- In most of the tables of data (Section 4.0), it is this Critical Level that appears in column 2.

- Unless otherwise indicated, the precision of the measurements reported here is given as ± 1.96 S_a (95% confidence level), where S_a is the standard deviation of the activity measurement.

- The value and standard deviation are reported with two significant figures using modified scientific notation, for example 0.032 is expressed as 3.2E-02.

The lower limits of detection (LLD) of all radionuclides in the various sample media are shown in Tables A.01 to A.11. The Annual Dose is to the Representative Person. The LLDs are based on typical data. Decay of radionuclides is accounted for in the LLD calculations except for H-3 and C-14 (long half-lives). The major assumptions are that the sample is taken at one kilometre from the point of emissions and that the level is maintained for the year. Milk is assumed to be from a cow pastured at 1.5 km, fish and lobster are caught at the Condenser Cooling Water (CCW) outlet and sediment, dulse, seawater and clams are collected at Dipper Harbour.

The CSA recommends, where technically feasible, that all measurements achieve LLDs less than that which would result in a dose of 5 μSv to the Representative Person. Most radionuclides pass this criterion. The major exceptions are noble gases. Detection of this group is through TLD measurements (20 μSv dose to the Representative Person at the LLD). However, the noble gas spectrometer on the GEM allows for a much smaller LLD calculation. Other exceptions are Ba-140 in soil and sediment (10 to 12 μSv); Ru-106 in water and clams (7 to 17 μSv); Ce-144 in water (11 μSv); and I-131 in food (8 μSv). Effluent analyses show these radionuclides are not major components of emissions. Part of the QA process identifies those LLDs or activities that do not meet this target.

A1.01 Air

A1.01.01 Airborne Particulates

Typical LLDs are given for a 2400 m^3 sample that is counted for 5000 s. The LLDs are decay corrected to the midpoint between the start and end of sampling, except for the gross alpha/beta results which represent the long-lived activity present a few days after sample collection. Gross alpha/beta is for trending only.

A1.01.02 Airborne Radioiodines

A typical LLD for I-131 is approximately 9E-05 $Bq \cdot m^{-3}$ (for a 2400 m^3 sample, counted for 50 000 s), which is decay corrected to the midpoint between the start and end of sampling.

A1.01.03 Airborne Tritium

The LLD is approximately 1E-01 $Bq \cdot m^{-3}$ of air for a typical sample of 10 to 70 m^3 (counted for 100 min). Due to the long half-life and relatively short period of time between sampling and analysis, decay correction is not applied.

A1.01.04 Airborne Carbon-14

A typical LLD is approximately 4E-02 $Bq \cdot m^{-3}$ of air for a 30 m^3 sample (counted for 100 min). Due to the long half-life and relatively short period of time between sampling and analysis, decay correction is not applied.

A1.01.05 TLD

The LLD is about 20 μSv. For typical quarterly measurements in the region of 150-200 μSv, measurements can be made to ± 10% at the 95% confidence level.

Table A.01 Annual Dose at the LLD for Air

Nuclide	*LLD ($Bq{\cdot}m^{-3}$)*	*Dose at LLD (μSv)*	*Concentration That Gives 5 μSv ($Bq{\cdot}m^{-3}$)*
H-3	9.6E-02	4.8E-02	9.9E+00
C-14	4.0E-02	1.9E+00	1.0E-01
Cr-51	5.8E-04	3.2E-03	9.2E-01
Mn-54	7.8E-05	9.2E-02	4.3E-03
Fe-59	1.7E-04	6.1E-02	1.4E-02
Co-58	8.0E-05	3.5E-02	1.2E-02
Co-60	8.2E-05	1.7E+00	2.4E-04
Zn-65	1.9E-04	3.3E-01	2.9E-03
Kr-85	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Kr-85m	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Kr-87	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Kr-88	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Zr-95	1.3E-04	1.1E-01	6.2E-03
Nb-95	9.4E-05	9.9E-02	4.7E-03
Ru-103	7.4E-05	8.1E-03	4.5E-02
Ru-106	6.0E-04	1.0E+00	2.9E-03
Ag-110m	6.2E-05	2.2E-01	1.4E-03
I-131	8.4E-05	1.6E-01	2.5E-03
Xe-131m	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Xe-133	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Xe-133m	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Xe-135	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Xe-135m	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Xe-138	2.0E+01 μSv	2.0E+01	5.0E+00 μSv
Cs-134	6.4E-05	4.3E-01	7.4E-04
Cs-137	6.6E-05	1.6E+00	2.0E-04
Ba-140	4.8E-04	8.9E-02	2.7E-02
La-140	2.0E-04	2.5E-03	4.1E-01
Ce-141	7.6E-05	4.8E-03	7.9E-02
Ce-144	2.2E-04	2.7E-01	4.0E-03

A1.02 Milk

The LLDs in Table A.02 apply to the midpoint between the start and end of sampling for a 3.6 L sample counted for 50 000 s for gamma and a 6 mL sampled counted for 100 min for tritium.

Table A.02 Annual Dose at the LLD for Milk

Nuclide	*LLD ($Bq{\cdot}L^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq{\cdot}L^{-1}$)*
H-3	2.4E+01	3.1E-01	3.9E+02
Cr-51	6.4E-01	1.0E-02	3.1E+02
Mn-54	9.2E-02	1.9E-02	2.5E+01
Fe-59	2.2E-01	2.2E-01	5.1E+00
Co-58	9.0E-02	7.5E-02	6.0E+00
Co-60	1.1E-01	9.0E-01	6.1E-01
Zn-65	2.4E-01	5.3E-01	2.3E+00
Zr-95	1.6E-01	4.3E-02	1.8E+01
Nb-95	9.2E-02	1.2E-01	3.8E+00
Ru-103	8.4E-02	1.9E-02	2.2E+01
Ru-106	7.8E-01	2.7E+00	1.4E+00
Ag-110m	8.2E-02	5.5E-02	7.4E+00
I-131	9.4E-02	1.8E+00	2.6E-01
Cs-134	7.8E-02	3.1E-01	1.3E+00
Cs-137	9.8E-02	3.1E-01	1.6E+00
Ba-140	3.4E-01	2.1E-01	8.1E+00
La-140	1.1E-01	8.9E-02	6.1E+00
Ce-141	1.2E-01	3.0E-02	2.1E+01
Ce-144	5.0E-01	9.5E-01	2.6E+00

A1.03 Water

The LLDs in Table A.03 apply to the midpoint between the start and end of sampling for a 3.6 L sample counted for 5000 s for gamma and a 6 mL sampled counted for 100 min for tritium. Alpha/beta results (a 100-500 mL sample counted for 100 min) represent the long-lived activity present several days after sample collection.

The LLDs are based on typical data for precipitation water. Since decay of radionuclides is accounted for in the LLD calculations, well water and other water sample types will have lower LLDs. The major assumptions are that the sample is taken at one kilometre from the point of emissions, that the level is maintained for the year and the sample type is the major source of drinking water. Obviously, this is not the case but it gives a simple “worst case” that is easy to monitor and calculate.

Table A.03 Annual Dose at the LLD for Water

Nuclide	*LLD ($Bq \cdot L^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq \cdot L^{-1}$)*
H-3	2.4E+01	3.4E-01	3.6E+02
Cr-51	5.4E+01	7.4E-02	3.7E+03
Mn-54	5.0E-01	1.7E-01	1.4E+01
Fe-59	1.3E+00	9.1E-01	7.1E+00
Co-58	5.6E-01	3.7E-01	7.5E+00
Co-60	4.6E-01	4.5E+00	5.1E-01
Zn-65	1.1E+00	2.3E+00	2.4E+00
Zr-95	9.8E-01	4.0E-01	1.2E+01
Nb-95	6.8E-01	4.0E-01	8.5E+00
Ru-103	6.4E-01	1.8E-01	1.8E+01
Ru-106	4.6E+00	1.7E+01	1.4E+00
Ag-110m	4.6E-01	6.2E-01	3.7E+00
I-131	2.4E+00	3.9E+00	3.1E+00
Cs-134	4.4E-01	4.8E+00	4.6E-01
Cs-137	5.2E-01	3.9E+00	6.6E-01
Ba-140	5.4E+00	2.7E+00	1.0E+01
La-140	2.2E+00	9.1E-01	1.2E+01
Ce-141	8.4E-01	3.4E-01	1.2E+01
Ce-144	2.4E+00	1.1E+01	1.1E+00

A1.04 Food

The LLDs in Table A.04 apply to the time of sample collection. Samples vary in size and are counted for 5000 s. The LLDs are based on typical data for garden vegetables.

Table A.04 Annual Dose at the LLD for Food

Nuclide	*LLD ($Bq \cdot kg^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq \cdot kg^{-1}$)*
Cr-51	3.0E+01	5.9E-02	2.5E+03
Mn-54	3.4E+00	1.3E-01	1.3E+02
Fe-59	7.8E+00	8.4E-01	4.6E+01
Co-58	3.6E+00	3.0E-01	6.0E+01
Co-60	3.8E+00	3.9E+00	4.9E+00
Zn-65	9.0E+00	2.2E+00	2.1E+01
Zr-95	6.2E+00	3.3E-01	9.4E+01
Nb-95	4.0E+00	3.6E-01	5.6E+01
Ru-103	3.8E+00	1.7E-01	1.1E+02
Ru-106	3.0E+01	1.3E+01	1.1E+01
Ag-110m	3.0E+00	4.7E-01	3.2E+01
I-131	1.0E+01	6.9E+00	7.6E+00
Cs-134	3.0E+00	3.6E+00	4.2E+00
Cs-137	3.4E+00	2.9E+00	6.0E+00
Ba-140	2.4E+01	3.5E+00	3.5E+01
La-140	9.4E+00	1.2E+00	4.0E+01
Ce-141	4.2E+00	1.9E-01	1.1E+02
Ce-144	1.4E+01	4.9E+00	1.4E+01

A1.05 Soil

The LLDs in Table A.05 apply to the time of sample collection. Samples of approximately 200 g are counted for 5000 s.

Table A.05 Annual Dose at the LLD for Soil

Nuclide	*LLD ($Bq \cdot kg^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq \cdot kg^{-1}$)*
Cr-51	4.0E+01	2.5E-01	7.9E+02
Mn-54	5.8E+00	1.2E+00	2.5E+01
Fe-59	1.2E+01	3.1E+00	1.9E+01
Co-58	5.0E+00	1.1E+00	2.2E+01
Co-60	5.8E+00	3.2E+00	9.1E+00
Zn-65	1.3E+01	1.7E+00	3.9E+01
Zr-95	1.0E+01	5.1E+00	9.9E+00
Nb-95	6.0E+00	9.9E-01	3.0E+01
Ru-103	4.8E+00	5.1E-01	4.7E+01
Ru-106	4.6E+01	1.9E+00	1.2E+02
Ag-110m	5.2E+00	2.7E+00	9.7E+00
I-131	6.8E+00	5.2E-01	6.6E+01
Cs-134	5.2E+00	1.5E+00	1.7E+01
Cs-137	5.6E+00	7.1E-01	3.9E+01
Ba-140	2.2E+01	1.1E+01	9.6E+00
La-140	7.2E+00	*	*
Ce-141	6.8E+00	1.2E-01	2.8E+02
Ce-144	2.4E+01	2.9E-01	4.2E+02
TLD	2.0E+01 µSv	2.0E+01	5.0E+00 µSv
*Dose for Ba-140 assumes equilibrium with La-140 (contribution from both)			

A1.06 Seawater

The LLDs in Table A.06 apply to the time of sample collection for a 3.6 L sample counted for 5000 s for gamma; and a 6 mL sampled counted for 100 min for tritium. The dose is small due to the simple facts that the frigid waters of the Bay of Fundy discourage immersion and salt water is not consumable.

Table A.06 Annual Dose at the LLD for Seawater

Nuclide	*LLD ($Bq{\cdot}L^{-1}$)*	*Dose at LLD (μSv)*	*Concentration That Gives 5 μSv ($Bq{\cdot}L^{-1}$)*
H-3	2.4E+01	9.2E-11	1.3E+12
Cr-51	2.2E+00	2.6E-10	4.3E+10
Mn-54	2.8E-01	7.9E-10	1.8E+09
Fe-59	6.2E-01	2.6E-09	1.2E+09
Co-58	2.8E-01	9.1E-10	1.5E+09
Co-60	3.2E-01	2.4E-09	6.8E+08
Zn-65	6.8E-01	1.3E-09	2.6E+09
Zr-95	5.2E-01	1.2E-09	2.2E+09
Nb-95	3.0E-01	9.3E-10	1.6E+09
Ru-103	2.8E-01	4.4E-10	3.2E+09
Ru-106	2.6E+00	1.6E-09	8.3E+09
Ag-110m	2.6E-01	2.3E-09	5.7E+08
I-131	3.6E-01	2.3E-09	8.0E+08
Cs-134	2.6E-01	1.3E-09	1.0E+09
Cs-137	3.0E-01	5.2E-10	2.9E+09
Ba-140	1.2E+00	2.6E-08	2.4E+08
La-140	4.6E-01	*	*
Ce-141	4.0E-01	1.1E-10	1.8E+10
Ce-144	1.6E+00	2.7E-10	3.0E+10
* Dose for Ba-140 assumes equilibrium with La-140 (contribution from both)			

A1.07 Clams

Typical LLDs are given in Table A.07 for the edible portions of clams, decay corrected to the time of sample collection. Samples of varying size are counted for 5000 s. The major assumptions are that the sample is taken at Dipper Harbour and that the level is maintained for the year.

Table A.07 Annual Dose at the LLD for Clams

Nuclide	*LLD ($Bq \cdot kg^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq \cdot kg^{-1}$)*
Cr-51	5.0E+01	2.0E-02	1.3E+04
Mn-54	7.0E+00	6.5E-02	5.4E+02
Fe-59	1.4E+01	3.1E-01	2.3E+02
Co-58	7.2E+00	1.3E-01	2.7E+02
Co-60	6.4E+00	1.8E+00	1.8E+01
Zn-65	1.4E+01	8.8E-01	7.8E+01
Zr-95	1.2E+01	1.5E-01	3.7E+02
Nb-95	6.6E+00	1.4E-01	2.3E+02
Ru-103	6.0E+00	5.5E-02	5.5E+02
Ru-106	5.8E+01	6.5E+00	4.5E+01
Ag-110m	5.8E+00	2.2E-01	1.3E+02
I-131	7.2E+00	9.5E-01	3.8E+01
Cs-134	6.6E+00	1.6E+00	2.1E+01
Cs-137	6.8E+00	1.5E+00	2.2E+01
Ba-140	2.4E+01	7.7E-01	1.6E+02
La-140	9.4E+00	2.2E-01	2.1E+02
Ce-141	7.4E+00	8.0E-02	4.6E+02
Ce-144	3.2E+01	2.6E+00	6.3E+01

A1.08 Fish

Typical LLDs are given in Table A.08 for the edible portions of fish, decay corrected to the time of sample collection. Samples of varying size are counted for 5000 s.

Table A.08 Annual Dose at the LLD for Fish

Nuclide	*LLD ($Bq \cdot kg^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq \cdot kg^{-1}$)*
Cr-51	1.3E+01	1.6E-02	4.0E+03
Mn-54	1.5E+00	5.1E-02	1.5E+02
Fe-59	3.8E+00	2.8E-01	6.8E+01
Co-58	1.5E+00	9.2E-02	8.0E+01
Co-60	1.4E+00	1.3E+00	5.4E+00
Zn-65	3.0E+00	7.0E-01	2.2E+01
Zr-95	2.2E+00	1.1E-01	1.0E+02
Nb-95	1.4E+00	1.1E-01	6.6E+01
Ru-103	1.5E+00	4.9E-02	1.5E+02
Ru-106	1.1E+01	4.4E+00	1.2E+01
Ag-110m	1.2E+00	1.7E-01	3.5E+01
I-131	7.8E+00	1.3E+00	3.1E+01
Cs-134	1.0E+00	1.2E+00	4.5E+00
Cs-137	1.4E+00	1.0E+00	7.1E+00
Ba-140	1.0E+01	7.8E-01	6.4E+01
La-140	4.6E+00	2.4E-01	9.6E+01
Ce-141	1.8E+00	6.0E-02	1.5E+02
Ce-144	5.8E+00	1.6E+00	1.8E+01

A1.09 Lobster

Typical LLDs are given in Table A.09 for the edible portions of lobster, decay corrected to the time of sample collection. Samples of varying size are counted for 5000 s.

Table A.09 Annual Dose at the LLD for Lobster

Nuclide	*LLD ($Bq \cdot kg^{-1}$)*	*Dose at LLD (μSv)*	*Concentration That Gives 5 μSv ($Bq \cdot kg^{-1}$)*
Cr-51	3.0E+01	1.3E-02	1.2E+04
Mn-54	2.8E+00	2.1E-02	6.7E+02
Fe-59	9.0E+00	1.5E-01	2.9E+02
Co-58	3.2E+00	6.7E-02	2.4E+02
Co-60	3.8E+00	5.4E-01	3.5E+01
Zn-65	7.8E+00	3.4E-01	1.2E+02
Zr-95	5.4E+00	6.8E-02	4.0E+02
Nb-95	4.4E+00	9.0E-02	2.4E+02
Ru-103	4.0E+00	3.1E-02	6.4E+02
Ru-106	3.0E+01	2.4E+00	6.3E+01
Ag-110m	3.4E+00	8.8E-02	1.9E+02
I-131	1.7E+01	3.3E+00	2.6E+01
Cs-134	2.8E+00	6.4E-01	2.2E+01
Cs-137	3.4E+00	4.5E-01	3.8E+01
Ba-140	3.4E+01	1.2E+00	1.4E+02
La-140	1.2E+01	4.2E-01	1.4E+02
Ce-141	4.4E+00	4.3E-02	5.1E+02
Ce-144	1.3E+01	8.7E-01	7.4E+01

A1.10 Dulse

Typical LLDs are given in Table A.10 for dulse, decay corrected to the time of sample collection. Samples of varying size are counted for 5000 s. The major assumptions are that the sample is taken at Dipper Harbour and that the level is maintained for the year.

Table A.10 Annual Dose at the LLD for Dulse

Nuclide	*LLD ($Bq \cdot kg^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq \cdot kg^{-1}$)*
Cr-51	2.0E+01	2.4E-03	4.2E+04
Mn-54	3.4E+00	7.4E-03	2.3E+03
Fe-59	8.0E+00	4.4E-02	9.0E+02
Co-58	3.4E+00	1.5E-02	1.1E+03
Co-60	3.4E+00	2.1E-01	7.9E+01
Zn-65	8.2E+00	1.1E-01	3.9E+02
Zr-95	6.6E+00	1.6E-02	2.1E+03
Nb-95	3.6E+00	1.9E-02	9.5E+02
Ru-103	2.8E+00	7.1E-03	2.0E+03
Ru-106	2.6E+01	6.7E-01	2.0E+02
Ag-110m	3.0E+00	2.3E-02	6.4E+02
I-131	5.4E+00	3.0E-01	9.1E+01
Cs-134	2.8E+00	1.9E-01	7.3E+01
Cs-137	3.2E+00	1.3E-01	1.2E+02
Ba-140	1.6E+01	1.6E-01	4.9E+02
La-140	5.4E+00	2.8E-02	9.7E+02
Ce-141	3.4E+00	8.8E-03	1.9E+03
Ce-144	1.4E+01	2.4E-01	2.9E+02

A1.11 Sediment

The LLDs in Table A.11 apply to the time of sample collection. Samples weighing approximately 200 g are counted for 5000 s. The major assumptions are that the sample is taken at Dipper Harbour and that the level is maintained for the year.

Table A.11 Annual Dose at the LLD for Sediment

Nuclide	*LLD ($Bq\cdot kg^{-1}$)*	*Dose at LLD (µSv)*	*Concentration That Gives 5 µSv ($Bq\cdot kg^{-1}$)*
Cr-51	1.7E+01	2.1E-01	4.2E+02
Mn-54	2.8E+00	7.9E-01	1.8E+01
Fe-59	6.2E+00	2.3E+00	1.3E+01
Co-58	2.6E+00	8.4E-01	1.5E+01
Co-60	2.8E+00	2.3E+00	6.1E+00
Zn-65	6.8E+00	1.3E+00	2.7E+01
Zr-95	4.8E+00	3.9E+00	6.1E+00
Nb-95	3.0E+00	7.2E-01	2.1E+01
Ru-103	2.4E+00	3.9E-01	3.1E+01
Ru-106	2.0E+01	1.5E+00	6.6E+01
Ag-110m	2.2E+00	2.1E+00	5.3E+00
I-131	3.4E+00	4.7E-01	3.6E+01
Cs-134	2.0E+00	1.2E+00	8.7E+00
Cs-137	2.8E+00	6.2E-01	2.3E+01
Ba-140	1.2E+01	1.0E+01	5.8E+00
La-140	3.8E+00	*	*
Ce-141	3.0E+00	8.9E-02	1.7E+02
Ce-144	1.1E+01	2.1E-01	2.7E+02
gamma meter	0.01 $\mu Sv\cdot h^{-1}$	3.0E+00	1.7E-02
* Dose for Ba-140 assumes equilibrium with La-140 (contribution from both)			

Appendix B: Sample Collection and Analytical Techniques

B1 Analytical Techniques

All environmental samples are analysed at the Health Physics Fredericton Laboratory. The following pages provide a general summary of the analytical techniques used in the laboratory. Sample collection, preparation and analysis are briefly described, but can be found in detail in the laboratory procedures.

The major analytical techniques and the instruments used in routine environmental analyses are summarised in the Table B.01.

Table B.01 Summary of Analytical Techniques

Analytical Technique	Instrumentation
Gamma Spectroscopy	Canberra 24% efficient* intrinsic, Ge detector in an Applied Physical Technology 10 cm graded lead cave; Canberra S-100 MCA
Liquid Scintillation (tritium and C-14)	Beckman LS 6000TA Liquid Scintillation Counter
Gross Alpha/Beta (Wet Chemical Analysis for Sr-89 and Sr-90)	Tennelec LB-5100 Alpha/Beta Counting System and Protean WPC 9550 Counting System
Gamma Surveys	Eberline Model FH 40G-10 low range gamma survey meter (range 10 $nSv \cdot h^{-1}$ to 1 $Sv \cdot h^{-1}$ for 30 keV to 3 MeV photons).
Thermoluminescent Dosimetry	Panasonic UD-7900U Automatic and UD-716AGL TLD Readers and UD-804A1 ($CaSO_4$) dosimeters

*relative to a 3x3 inch sodium iodide detector

In gamma spectroscopy analysis, all significant peaks in the spectrum are identified either by reference to a database library of about 150 radionuclides, or by manual reference to compilations of all known radionuclides. In addition, approximately 20 selected radionuclides are specifically searched for in every sample with the exception of Air Iodine samples in which only I-131 is selected. The selected radionuclides include those that are produced in PLNGS, and which would be readily detectable because of their abundance (high fission yield) and high branching ratios for gamma emissions. Naturally occurring gamma emitters, with the exception of Be-7, K-40 and Ac-228, are not included in this report. These excepted radionuclides are sometimes useful as general indicators of the consistency of the analytical techniques.

The peak search and analysis program SAMPO is used to process spectra. The library of radionuclides uses data of the Oak Ridge Laboratory. There are three categories of radionuclides evaluated:

1) selected nuclides of key fission and activation products
2) all other identified radionuclides, including natural radionuclides
3) detected energy peaks for which no identification can be readily made.

The three categories cover all possible eventualities in a spectral analysis and ensure that no significant radionuclides or photon energies will be overlooked.

The usefulness of gross alpha/beta analysis lies primarily in showing trends and determining whether more detailed analyses should be done. The reported alpha and beta values are assessed with respect to Am-241 and Sr-Y-90 calibration sources, respectively.

Wet chemical analysis for Sr-89,90 on GEM and LEM samples follows a method developed by Eichrom Industries Inc.[20] using a strontium specific chromatography resin. This method is similar to test method 05811-95 issued by the American Society of Tests and Materials (ASTM).

Liquid samples, other than milk, are acidified upon receipt to keep radionuclides from plating out on the container walls. Perishable samples are refrigerated or frozen.

B2 Sample Collection and Analysis

B2.01 Airborne Particulates

Airborne particulates are collected on a 47 mm diameter Gelman Type A glass fibre filter, through which air is drawn at about 60 $L \cdot min^{-1}$ for a 28 day continuous sample. The volume of air sampled (approximately 2400 m^3) is measured with an in-line integrating dry gas meter. Every month the filters are replaced and the used ones are returned to the laboratory for analysis. Sampling is, therefore, continuous throughout the year.

Air particulate filters are analysed by gamma spectroscopy as soon as possible after collection to ensure the detection of any short lived gamma emitters that may be present, and to minimise any decay corrections. Samples are counted for 5000 s on the Ge detector.

Approximately three days after the end of the sample collection interval, each filter is counted on one of the alpha beta counters for 100 minutes for the simultaneous determination of gross alpha and gross beta activities. Counting is delayed to allow for the decay of the short-lived radon progeny that would otherwise complicate the analysis.

If alpha/beta levels are detected at twice the normal level, further investigation is initiated by longer gamma counts or radiostrontium determinations.

If levels of Sr-89,90, indicating one percent of the weekly DRL, are detected in the chemical analysis of GEM filters, then the air monitoring station particulate filters are also to be analysed for these radionuclides.

B2.02 Airborne Radioiodines

Airborne radioiodines are collected in an activated charcoal cartridge placed downstream of the particulate filter. The cartridges are from F&J Specialty Products (TE3C 20x40 mesh TEDA). Approximately 2400 m^3 of air is sampled continuously over 28 days at a flow rate of about 60 $L\cdot min^{-1}$. The volume of air sampled is measured with an in-line integrating dry gas meter.

Iodine-131 is the major nuclide of interest on the charcoal cartridges. The cartridges are counted in groups of four for 50 000 s on the gamma spectrometer. Counts are performed as soon as possible after collection because of the relatively short-half life of I-131 (8 days). If radioiodines, believed to have originated from PLNGS, are detected, then the cartridges are re-analysed individually. Fission product radioiodines other than I-131, with much shorter half-lives (minutes to hours), decay before they reach the sample location or during the time the sample is being collected. If a significant release of radioiodines were noted from the station in this interval, the samples would be changed and analysed earlier to minimise errors from decay corrections.

B2.03 Airborne Tritium

Air is passed through a molecular sieve container (Advanced Specialty Gas Equipment type 13X sieve material) to extract water vapour from the sampled air. Sample volume is measured with a mass flow controller (MFC) (Alicat Scientific Inc. MC-1SLPM-0).

Sampling is continuous at each location throughout the year. Since the amount of water absorbed by the molecular sieve from a given volume of air depends upon absolute humidity, flow rates are adjusted with a MFC to avoid saturation of the sieve material and to ensure adequate sample collection.

For tritium analysis by liquid scintillation counting, 6 mL of water taken from the molecular sieve condensate is counted for 100 minutes.

B2.04 Airborne Carbon-14

An aquarium pump bubbles air through 2N NaOH (1 L), into which carbon dioxide and its C-14 component is absorbed. Carbon dioxide is regenerated from the resulting sodium carbonate by acidification of the 2N NaOH solution and then analysed for the determination of C-14 activity. The carbon dioxide is passed through a silica gel trap to remove moisture and tritium and then absorbed into the chemical Carbo-sorb® E until saturation is reached. After the addition of the scintillation cocktail Permafluor® E^+, the sample is analysed for 100 minutes by liquid scintillation counting.

B2.05 Environmental Gamma Radiation (TLD)

The environmental TLD is composed of three elements of calcium sulphate with lead filtration of 700 $mg \cdot cm^{-2}$. The badge is sealed in plastic, placed in a screw cap plastic container and suspended approximately 1 m above the ground for a period of three months. This arrangement measures the ambient gamma dose, whether it is from activity in the air, from the ground or cosmic in origin.

Readout is by a Panasonic Automatic Reader. For typical quarterly measurements in the region of 150-200 μSv, measurements can be made to ±10% at the 95% confidence level.

B2.06 Soil

Soil samples are collected in undisturbed locations away from nearby buildings or trees. Level areas with some vegetation are preferred. A representative sample (approximately 1.6 kg) of the top 25 mm of a 20 cm by 20 cm area of soil is placed in a disposable polyethylene container.

The soil is air dried overnight. If excessive moisture is present, the sample is dried on a disposable aluminum tray (at 100 °C). Composed organic matter and stones are removed. Approximately 0.25 kg of dry soil is counted by gamma spectroscopy for 5000 s.

B2.07 Food

Garden produce and berries, which are either collected or purchased, require no special preparation. The edible portion is put in a calibrated container and weighed. The sample is counted by gamma spectroscopy for 5000 s.

B2.08 Milk

A 4L sample is purchased and placed in a clean polyethylene container.

For gamma spectroscopy, a 3.6 L portion is measured into a marinelli beaker. Approximately 100 mL is distilled, and a 6 mL aliquot of the distillate is analysed for H-3 by liquid scintillation counting. Count times are 50 000 s for gamma spectroscopy and 100 min for tritium analysis.

B2.09 Water

A 4L sample of well water, pond water, lake water or surface runoff is collected in a clean polyethylene container.

A portion is removed for tritium analysis, and the remainder is acidified (15 mL of 70% nitric acid per 4 L sample). Of this, 3.6 L is measured into a marinelli beaker for gamma spectroscopy. After gamma analysis, well water samples (125-500 mL, depending upon the historical content of dissolved solids) are evaporated until dry on stainless steel planchets for gross alpha/beta analysis. For tritium analysis, a 6 mL aliquot is analysed by liquid scintillation counting. For gamma spectroscopy, the sample is counted for 5000 s. For tritium and gross alpha/beta analyses, samples are counted for 100 min. A level twice the normal level for alpha/beta will initiate further investigation by longer gamma counts and/or Sr-89,90 analyses.

Measurements of gross alpha and beta are made approximately two weeks after sample collection. This delay avoids analytical interference from radon progeny, which decay with a half-life of about 3.8 days. Naturally occurring radon and radon progeny are present in well waters everywhere and are known to reach elevated concentrations in many New Brunswick locations.

B2.10 Vegetation

The only vegetation types routinely collected and analysed are tree lichen (Spanish moss) and various ground mosses such as Cladonia and Lycopodium. They concentrate a wide range of radionuclides, both natural and man-made. This makes vegetation a sensitive indicator of radionuclides in the environment even though they are not identified in the pathway to humans.

About 25 g or more of each of the samples is collected and air-dried before analysis. No special preparation is required. The sample is placed in a calibrated container, weighed and counted by gamma spectroscopy for 5000 s.

B2.11 Precipitation

Various forms of precipitation are collected continuously throughout the year.

A portion is removed for tritium analysis and the remainder is acidified (15 mL of 70% nitric acid per 4 L sample). For gamma spectroscopy, 3.6 L is measured into a marinelli beaker and counted for 5000 s. For tritium analysis by liquid scintillation techniques, 6 mL is counted for 100 min.

B2.12 Sediment and Beach Surveys

Beach sediment samples are collected near the low tide mark, with preference being given to the top 10 mm of the fine sediment characteristic of tidal mud flats. A disposable polyethylene container is used to collect about 1 kg of sample. In addition, direct gamma radiation dose rate measurements are made at each sediment site using a FAG FH 40F2 low range gamma survey meter. The meter is held for one minute at a point one metre above the intertidal surface. After the sediment sample has been collected, this is repeated.

The sample is transferred to a disposable aluminum tray for drying at 80 °C. Dried, caked samples are broken into their original free granular form with a porcelain mortar and pestle and sieved through a 0.5 mm mesh to collect the fines for analysis (a 1 mm sieve is used for coarse sediments). Approximately 0.25 kg of dried sediment is counted by gamma spectroscopy for 5000 s.

B2.13 Seafood

The inshore fishery throughout the Maritimes has declined since the OERMP was started in 1982. Some of it has been closed to any kind of harvesting. However, species of local seafood are collected when available from local fishermen. Sampling focuses on fish, lobsters, aquaculture salmon and clams. Some of the areas where clam harvesting is prohibited are sampled with the permission of the Department of Fisheries and Ocean. Other seafood species are more mobile and can sometimes be found throughout the area: crab, periwinkles, scallops, herring, mackerel, dogfish, cod, haddock, sea urchin, mussels, and flounder. The severe restrictions placed on the inshore fishery as well as the depletion of stocks make many of these samples unavailable for periods of time sometimes spanning years. However, whenever they are available an effort is made to collect as many samples as possible. Approximately 0.5 kg of fresh seafood is collected per sample.

Approximately 0.25 kg of each sample is prepared for gamma spectroscopy. Lobsters are cooked first, and the edible meat is removed for analysis. Clams, periwinkles, and crab are analysed whole, with a yield factor applied to account for the mass of the inedible shell. Usually the edible portion of fish is analysed, although sometimes the whole fish is analysed. Samples are counted for 5000 s.

B2.14 Aquatic Plants

Dulse (*Rhodymenia palmata*), an edible seaweed which is commercially harvested in the area, is collected monthly when available. Other species of seaweed concentrate a wide range of radionuclides, both natural and man-made. This makes them sensitive indicators of radionuclides in the environment even though they are not identified in the pathway to humans.

A portion of the seaweed or dulse is put in a calibrated container and weighed. This is counted by gamma spectroscopy for 5000 s.

B2.15 Seawater

A 4 L sample is collected in a clean polyethylene container.

A portion is removed for tritium analysis and the remainder is acidified (15 mL of 70% nitric acid per 4 L sample). For gamma spectroscopy, 3.6 L is measured into a marinelli beaker and counted for 5000 s. For tritium analysis by liquid scintillation techniques, 6 mL is counted for 100 min.

If levels of Sr-89,90, indicating one percent of the monthly DRL, are detected in the chemical analysis of the LEM composite, then the seawater is also to be analysed for these radionuclides.

B2.16 Miscellaneous Samples

This category encompasses all of those samples collected that do not fall within the other categories. It is a mechanism by which the lab can track and evaluate media for potential inclusion in the program. It gives the program flexibility and freedom and encourages the scientific curiosity of laboratory staff. A few of the media types started out this way. As many as 50 samples per year are analysed, including deer liver, mud puddles, snow, sea urchin and mussels.

B2.17 Bore Holes

A 4 L sample of water is pumped out of the bore hole into a clean polyethylene container.

A portion is removed for tritium analysis and the remainder is acidified (15 mL of 70% nitric acid per 4 L sample). For gamma spectroscopy, 3.6 L is measured into a marinelli beaker and counted for 5000 s. For tritium analysis by liquid scintillation techniques, 6 mL is counted for 100 min.

B2.18 Parshall Flume

PLNGS staff collect a 4 L sample of water from the Parshall flume systems.

A portion is removed for tritium analysis, and the remainder is acidified (15 mL of 70% nitric acid per 4 L sample). Of this, 3.6 L is measured into a marinelli beaker for gamma spectroscopy. For tritium analysis, a 6 mL sample of water is counted for 100 min by liquid scintillation techniques. For gamma spectroscopy, the sample is counted for 5000 s.

B2.19 Hemlock Knoll Regional Sanitary Landfill

In December 1999, PLNGS began disposing of its non-active waste at the public landfill facility. A monitoring program was established prior to the first shipment. It includes sampling of water from the leachate, bore holes and various holding ponds; and dosimeter placement at key locations.

Although some extra precautions are observed due to the potential biohazard of some of these samples, they are analysed according to established procedures previously described.

Appendix C: Location Codes

A 5m	PLNGS Dry Fuel Storage Facility - 5 m NNE from perimeter fence
A 10m	PLNGS Dry Fuel Storage Facility - 10 m NNE from perimeter fence
A 15m	PLNGS Dry Fuel Storage Facility - 15 m NNE from perimeter fence
A 20m	PLNGS Dry Fuel Storage Facility – 20 m NNE from perimeter fence
A 25m	PLNGS Dry Fuel Storage Facility – 25 m NNE from perimeter fence
A 50m	PLNGS Dry Fuel Storage Facility – 50 m NNE from perimeter fence
A 75m	PLNGS Dry Fuel Storage Facility – 75 m NNE from perimeter fence
A 100m	PLNGS Dry Fuel Storage Facility – 100 m NNE from perimeter fence
A 118m	PLNGS Dry Fuel Storage Facility – 118 m NNE from perimeter fence
A01R	Bocabec – GPS Reading – L 45˚ 10.111N, Lo 67˚ 0.378 W
A02R	Bocabec – field across from A01R
A03R	Bocabec – inter-tidal zone
A04	Bayside – Farm
A05R	Letete
A06	Digdeguash
A07	Beaver Harbour
A08	Back Bay
A09	Chamcook
A10R	Grand Manan
A11	Oak Bay / Waweig

A12	St. Andrews
A13R	St. Andrews environmental monitoring station
A15	Deer Island
A20	Campobello Island
AECL	Atomic Energy of Canada Ltd., Chalk River (QA)
ANA	Eckert & Ziegler Analytics (QA)
B 5m	PLNGS Dry Fuel Storage Facility – 5 m WNW from perimeter fence
B 10m	PLNGS Dry Fuel Storage Facility – 10 m WNW from perimeter fence
B 15m	PLNGS Dry Fuel Storage Facility – 15 m WNW from perimeter fence
B 20m	PLNGS Dry Fuel Storage Facility – 20 m WNW from perimeter fence
B 25m	PLNGS Dry Fuel Storage Facility – 25 m WNW from perimeter fence
B 50m	PLNGS Dry Fuel Storage Facility – 50 m WNW from perimeter fence
B 75m	PLNGS Dry Fuel Storage Facility – 75 m WNW from perimeter fence
B 100m	PLNGS Dry Fuel Storage Facility – 100 m WNW from perimeter fence
B 150m	PLNGS Dry Fuel Storage Facility – 150 m WNW from perimeter fence
B 200m	PLNGS Dry Fuel Storage Facility – 200 m WNW from perimeter fence
B01	New River Beach - inter-tidal zone
B02	Pocologan
B03	New River Beach - park
B04	New River Harbour to Pocologan Harbour

B10	Pennfield
BAXR	Baxter's Dairy
BB	PLNGS – Boiler Blow-down
BD	Belledune GS
C01	Lepreau Harbour – intertidal zone
C03	Lepreau
CC	Coleson Cove GS
CCW	PLNGS – Condenser Cooling Water Duct
CH	Chatham GS
COG	Kinectrics (CANDU Owners Group)
D01	Little Lepreau Basin - inter-tidal zone (remnants of clam shack)
D02	Little Lepreau
D03	Little Lepreau – GPS Reading – L 45˚ 08.030 N , Lo 66˚ 27.686 W
D04	Little Lepreau Basin – inter-tidal zone (remnants of boat wreck)
DH	Dalhousie GS
DOE	US Department of Energy (QA)
DUMP	PLNGS – onsite landfill
DWC	PLNGS – drinking water fountains
E01	Maces Bay –GPS Reading–L 45˚ 06.306 N, Lo 66˚ 28.651 W
E02	Maces Bay – Fundy Senior Citizens Centre
E03	Maces Bay – inter-tidal zone

E04	Maces Bay Cemetery
E05	Fundy Shores Elementary School – outside (Thompson/Trynor's Field)
E06	Fundy Shores Elementary School – inside
E07	Near intersection of route 790, Maces Bay Rd. and County Line Rd.
E11	28 Ridge Rd., Dipper Harbour
E12	22 Ridge Rd., Dipper Harbour
E13	16 Ridge Rd., Dipper Harbour
E14	10 Ridge Rd., Dipper Harbour
E15	4 Ridge Rd., Dipper Harbour
EDU	Edutech Enterprises
EPA	US Environmental Protection Agency (QA)
ERA	Environmental Resource Associates
F01	Welch Cove–GPS Reading–L 45˚ 04.782N, Lo 66˚ 27.986 W
F02	Welch Cove – inter-tidal zone
F03	190 Welch Cove Rd., Maces Bay
F04	195 Welch Cove Rd., Maces Bay
F05	181 Ridge Rd., Maces Bay
F06	132 Ridge Rd., Maces Bay
F07	68 Ridge Rd., Maces Bay
G01	Indian Cove – inter-tidal zone
G02	Point Lepreau – lighthouse

G03	offshore – within 2 km of Point Lepreau lighthouse
G04	PLNGS – inter-tidal zone 1 km south of CCW out-fall
GEM	PLNGS – Gaseous Effluent Monitor
GL	Grand Lake GS
H01	Duck Cove – duck pond
H02	offshore – close to PLNGS condenser cooling water out-fall
H03	Duck Cove - inter-tidal zone
H04	PLNGS – across the road from old site of Information Centre building
H05	PLNGS - start of nature trail near old site of Information Centre trailers
HS	Hanson Stream Reservoir
I00	PLNGS SRWMF Phase 1– general site area
I01	PLNGS SRWMF Phase 1
I02	PLNGS SRWMF Phase 2
I03	PLNGS SRWMF Phase 2 – general site area
I04	SRWMF Phase 3
I05	SRWMF Phase 3, General Site Area
I10A	PLNGS SRWMF Phase 1 Bore Hole A (BHA)
I10B	PLNGS SRWMF Phase 1 Bore Hole B (BHB)
I10C	PLNGS SRWMF Phase 1 Bore Hole C (BHC)
I10D	PLNGS SRWMF Phase 1 at I01 Barn (Shallow Bore Hole)
I10E	PLNGS SRWMF Phase 1 at I01 Barn (Deep Bore Hole)

I10F	PLNGS SRWMF Phase 1 Bore Hole southeast from C structure
I10G	FUTURE BORE HOLE
I10H	FUTURE BORE HOLE
I10I	FUTURE BORE HOLE
I11A	PLNGS SRWMF Phase 1 - south fence (east side)
I11B	PLNGS SRWMF Phase 1 - south fence (centre)
I11C	PLNGS SRWMF Phase 1 - south fence (west side)
I11D	PLNGS SRWMF Phase 1 - west fence (south side)
I11E	PLNGS SRWMF Phase 1- west fence (centre)
I11F	PLNGS SRWMF Phase 1 - west fence (north side)
I11G	PLNGS SRWMF Phase 1 - north fence (west side)
I11H	PLNGS SRWMF Phase 1 - north fence (centre)
I11I	PLNGS SRWMF Phase 1 - north fence (east side)
I11J	PLNGS SRWMF Phase 1 - east fence (north side)
I11K	PLNGS SRWMF Phase 1 - east fence (centre)
I11L	PLNGS SRWMF Phase 1 - east fence (south side)
I11M	SRWMF Phase 1 ext, Fence W-N
I11N	SRWMF Phase 1 ext, Fence W-NN
I11O	SRWMF Phase 1 ext, Fence N-W
I11P	SRWMF Phase 1 ext, Fence N-C
I11Q	SRWMF Phase 1 ext, Fence N-E

I11S	SRWMF Phase 1 ext, Fence E-NN
I11T	SRWMF Phase 1 ext, Fence E-N
I1A1	PLNGS SRWMF Phase 1 – Cell 1A1
I1A2	PLNGS SRWMF Phase 1 – Cell 1A2
I20A	PLNGS SRWMF Phase 2 – well #4 (shallow) BH4
I20B	PLNGS SRWMF Phase 2 – well #4 (deep) BH4
I20C	PLNGS SRWMF Phase 2 - well #7 (shallow) BH7
I20D	PLNGS SRWMF Phase 2 - well #7 (deep) BH7
I20E	PLNGS SRWMF Phase 2 – well #6 (shallow) BH6
I20F	PLNGS SRWMF Phase 2 - well #6 (deep) BH6
I20G	PLNGS SRWMF Phase 2 – well #5 (shallow) BH5
I20H	PLNGS SRWMF Phase 2 – well #5 (deep) BH5
I20I	PLNGS SRWMF Phase 2 – well #2 (shallow) BH2
I20J	PLNGS SRWMF Phase 2 - well #2 (deep) BH2
I20K	PLNGS SRWMF Phase 2 - well #3 (shallow) BH3
I20L	PLNGS SRWMF Phase 2 – well #3 (deep) BH3
I20M	PLNGS SRWMF Phase 2 – well #1 (shallow) BH1
I20N	PLNGS SRWMF Phase 2 – well #1 (deep) BH1
I20P	PLNGS SRWMF Phase 2 – north from bore hole 1
I20Q	PLNGS SRWMF Phase 2 – south from bore hole 2 (shallow)
I20S	PLNGS SRWMF Phase 2 – south from bore hole 2 (deep)

I20T	PLNGS SRWMF Phase 2 – north from bore hole 2
I20U	PLNGS SRWMF Phase 2 – well #8 shallow (BH8)
I20V	PLNGS SRWMF Phase 2 – well #8 deep (BH8)
I20W	SRWMF Phase 2, Middle NE Shallow
I21A	PLNGS SRWMF Phase 2 – Periphery – south fence (east side)
I21B	PLNGS SRWMF Phase 2 - Periphery – south fence (centre)
I21C	PLNGS SRWMF Phase 2 - Periphery – south fence (west side)
I21D	PLNGS SRWMF Phase 2 - Periphery – west fence (south side)
I21E	PLNGS SRWMF Phase 2- Periphery - west fence (centre)
I21F	PLNGS SRWMF Phase 2 - Periphery - west fence (north side)
I21G	PLNGS SRWMF Phase 2 – Periphery – north fence (west side)
I21H	PLNGS SRWMF Phase 2 - Periphery – north fence (centre)
I21I	PLNGS SRWMF Phase 2 - Periphery – north fence (east side)
I21J	PLNGS SRWMF Phase 2 – Periphery – east fence (north side)
I21K	PLNGS SRWMF Phase 2 – Periphery – east fence (centre)
I21L	PLNGS SRWMF Phase 2 - Periphery – east fence (south side)
I30A	SRWMF Phase 3, Well 1
I30B	SRWMF Phase 3, Well 2 Shallow
I30C	SRWMF Phase 3, Well 2 Deep
I30D	SRWMF Phase 3, Well 3
I30E	SRWMF Phase 3, Well 4

I30F	SRWMF Phase 3, Well 5 Shallow	**I31Q**	SRWMF Phase 3, Fence E-WC
I30G	SRWMF Phase 3, Well 5 Deep	**I31S**	SRWMF Phase 3, Fence E-W
I30H	SRWMF Phase 3, Well 6	**I31T**	SRWMF Phase 3, Fence E-WW
I30I	SRWMF Phase 3, Well 7	**I70**	PLNGS – woods between plant & SRWMF
I30J	SRWMF Phase 3, Well 8 Shallow	**I71**	PLNGS - Near Plant Monitoring Well MW01-10, northeast from RB
I30K	SRWMF Phase 3, Well 8 Deep	**I75**	PLNGS – north 73° east, 85 m from the stack (on pole)
I31A	SRWMF Phase 3, Fence S-E	**I86**	PLNGS – 2nd pole from SRWMF driveway heading toward outer gate
I31B	SRWMF Phase 3, Fence S–C	**I87**	PLNGS –3rd pole from SRWMF driveway heading toward outer gate
I31C	SRWMF Phase 3, Fence S-W	**I88**	PLNGS – 4th pole from SRWMF driveway heading toward outer gate
I31D	SRWMF Phase 3, Fence W-SS	**I89**	PLNGS -5th pole from SRWMF driveway heading toward outer gate
I31E	SRWMF Phase 3, Fence W-S	**I90**	At distribution line on west side of Point Lepreau Rd.
I31F	SRWMF Phase 3, Fence W-SC	**I91**	100 m north of distribution line on west side of Point Lepreau Rd.
I31G	SRWMF Phase 3, Fence W-NC	**I92**	200 m north of distribution line on west side of Point Lepreau Rd.
I31H	SRWMF Phase 3, Fence W-N	**I93**	300 m north of distribution line on west side of Point Lepreau Rd.
I31I	SRWMF Phase 3, Fence W-NN	**I94**	400 m north of distribution line on west side of Point Lepreau Rd.
I31J	SRWMF Phase 3, Fence N-W	**I95**	500 m north of distribution line on west side of Point Lepreau Rd.
I31K	SRWMF Phase 3, Fence N-E	**I96**	on the old Dupont warning sign at the end of the old “dynamite road”
I31L	SRWMF Phase 3, Fence N-C	**I97**	on the west side of the clearing at the end of the old “dynamite road”
I31M	SRWMF Phase 3, Fence E-NN	**I98**	PLNGS – north of SRWMF PHASE 2 (200 m north of transmission line)
I31N	SRWMF Phase 3, Fence E-N	**I99**	PLNGS – north of SRWMF PHASE 2 – (100 m north of transmission
I31P	SRWMF Phase 3, Fence E-NC	**IAEA**	International Atomic Energy Agency (QA)

J00	PLNGS – south, 180 m from the stack (on fence)
J01	PLNGS - Near Plant Monitoring Well MW01-1, near surge shaft
J02A	PLNGS - Near Plant Monitoring Well MW01-2 (shallow), SSE from
J02B	PLNGS - Near Plant Monitoring Well MW01-2 (deep), SSE from RB
J20	PLNGS – south 19° east, 115 m from the stack (on fence)
J35	PLNGS – south 34° east, 135 m from the stack (on sign)
J70	PLNGS – south 69° east, 70 m from the stack (on pole)
K00	PLNGS - Near Plant Monitoring Well MW01-3 south from RB
K01	PLNGS – 95 m west of south gate leading to the lighthouse
K02	PLNGS Cooling Water Pump-house – east fence near surge shaft
K03	PLNGS - Near Plant Monitoring Well MW01-4 SSW from RB
K03C	PLNGS sewage lagoon (chlorine contact tank)
K03E	PLNGS inactive drainage (east lagoon)
K03W	PLNGS inactive drainage (west lagoon)
K04	PLNGS - Near Plant Monitoring Well MW01-5, WSW from RB
K10	Firing Range
KDRP	KD Radpro
L01	PLNGS – site of old cement plant
L02	PLNGS – switchyard
L03	PLNGS – outer security building (main gate)
L04	PLNGS – construction stores

L05	PLNGS - Near Plant Monitoring Well MW01-6, WNW from RB
L06	PLNGS - Near Plant Monitoring Well MW01-7, paved staff parking
L07	PLNGS - Near Plant Monitoring Well MW01-8, construction parking
L08	PLNGS - Near Plant Monitoring Well MW01-9, N beyond fire
L09A	MW05-1, fire fighter training area
L09B	MW05-2, fire fighter training area
L09C	MW05-3, fire fighter training area
L09D	MW05-4, fire fighter training area
L09E	MW05-5, fire fighter training area
L10A	Landfill SW05-1
L10B	Landfill SW05-2
L10C	Landfill SW05-3
L10D	Landfill SW05-4
L10E	Landfill SW05-5
L10F	Landfill SW05-6
L10G	Landfill Seep
L11A	Landfill MW6
L11B	Landfill MW7
L11C	Landfill MW8
L11D	Landfill MW9
L11E	Landfill MW10

L11F	Landfill MW11
L11G	Landfill MW12
L11H	Landfill MW13
LAB	Fredericton – Health Physics Laboratory
LEM	PLNGS – Liquid Effluent Monitor
M02	PLNGS – Administration Building (2nd floor)
MISC	Miscellaneous locations
MQ	Mactaquac GS
N01	Dipper Harbour – GPS Reading – L 45˚ 05.399 N, Lo 66˚ 25.154 W
N02	Dipper Harbour – GPS Reading – L 45˚ 06.106 N, Lo 66˚ 24.949 W
N03	Dipper Harbour – GPS Reading – L 45˚ 05.551 N, Lo 66˚ 25.449 W
N04	Dipper Harbour – intertidal zone
N05	Dipper Harbour – beach behind restaurant
N06	Dipper Harbour – offshore
NTS	Nuclear Technology Services Inc. (QA)
P01	Chance Harbour – GPS Reading – L 45˚ 07.494 N, Lo 66˚ 21.456 W
P02	Little Dipper Harbour
P03	Liberty Hill – GPS Reading – L 45˚ 07.043 N, Lo 66˚ 21.498
P04	Round Meadow Farm
P05	Chance Harbour – 2 km offshore
PLNGS	PLNGS – general

Q01R	Lorneville
RPB	Radiation Protection Bureau, Health Canada (QA)
RPC	Research and Productivity Council
S00	Saint John and surrounding area
S10	Hammond River
SPL	Spruce Lake reservoir
TAYR	Taymouth
X03R	Fredericton - Chestnut Complex lab
X04R	Fredericton – reference seafood
X05R	Fredericton – reference milk test
X06R	West of Fredericton (Silverwood)
X10	Fredericton Junction – Atlantic Dairy Institute
X12	York Mills
Y#####	Hemlock Knoll Regional Sanitary Landfill

Appendix D: Abbreviations

CCW	Condenser Cooling Water
CL	Critical Level
CNSC	Canadian Nuclear Safety Commission
COG	CANDU Owners Group
CSA	Canadian Standards Association
DRL	Derived Release Limit
FWHM	Full Width Half Maxima
GEM	Gaseous Effluent Monitor
IAEA	International Atomic Energy Agency
ISO	International Organization for Standardization
LEM	Liquid Effluent Monitor
LLD	Lower Limit of Detection
LSC	Liquid Scintillation Counter
MFC	Mass Flow Controller
NBEMO	New Brunswick Emergency Measures Organization
NIST	National Institute of Standards and Technology
NRC	National Research Council
NTS	Nuclear Technology Services
OERMP	Operational Environmental Radiation Monitoring Program
PICA	Problem Identification and Corrective Action
PLNGS	Point Lepreau Nuclear Generating Station
QA	Quality Assurance
QC	Quality Control
REPD	Radiation and Environmental Protection Division
RPB	Radiation Protection Bureau
SEA	Significant Environmental Aspect
SRWMF	Solid Radioactive Waste Management Facility
TLD	Thermoluminescent Dosimeter
USDOE	United States Department of Energy